



Overview of available *post mortem* data: JET's main chamber

Presenter: *I. Jepu*

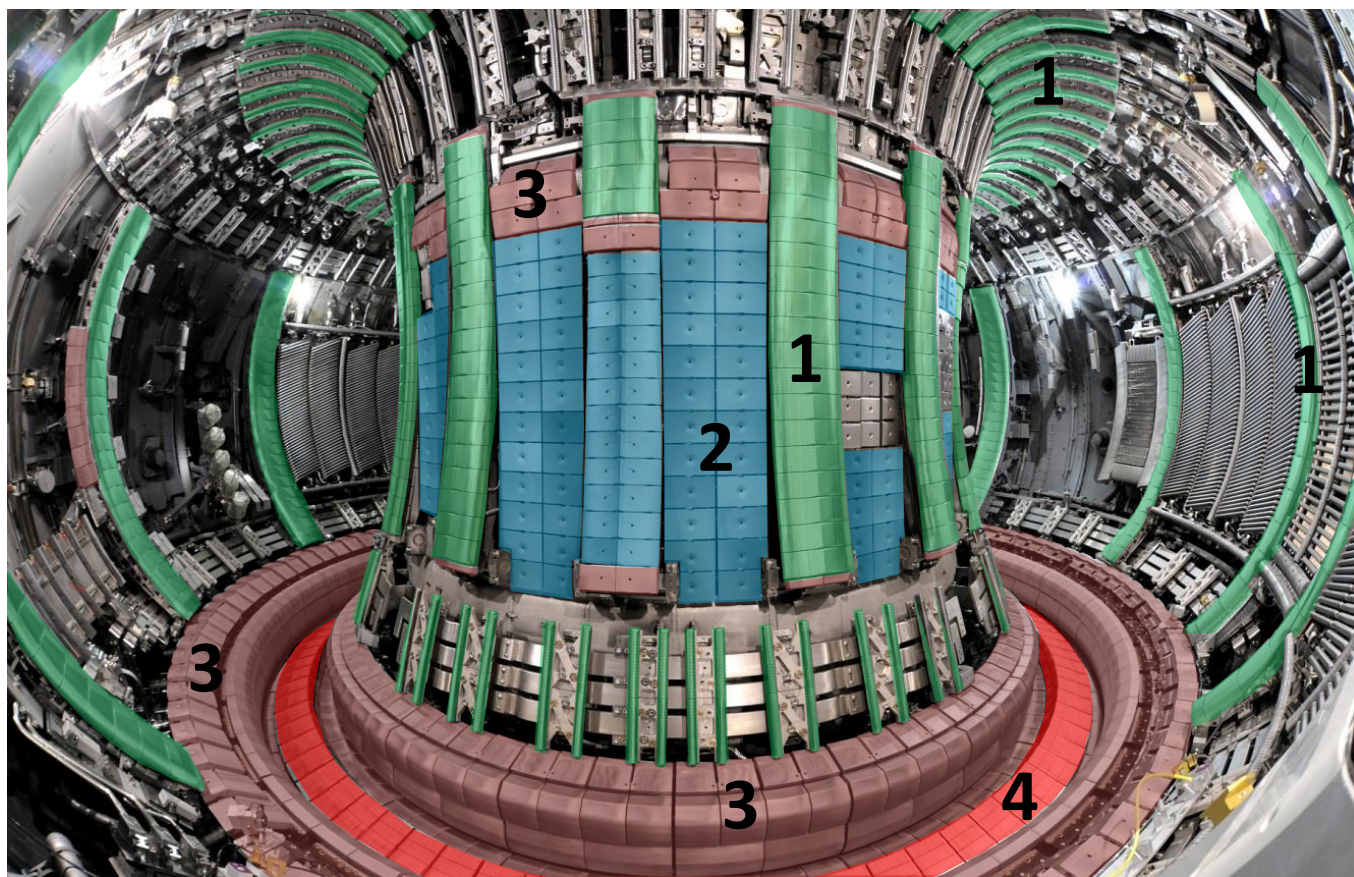
Material: *JET Contributors*



This work has been carried out within the framework of the EUROfusion Consortium, funded by the European Union via the Euratom Research and Training Programme (Grant Agreement No 101052200 — EUROfusion). Views and opinions expressed are however those of the author(s) only and do not necessarily reflect those of the European Union or the European Commission. Neither the European Union nor the European Commission can be held responsible for them.



Overview – JET's PFCs



Main chamber

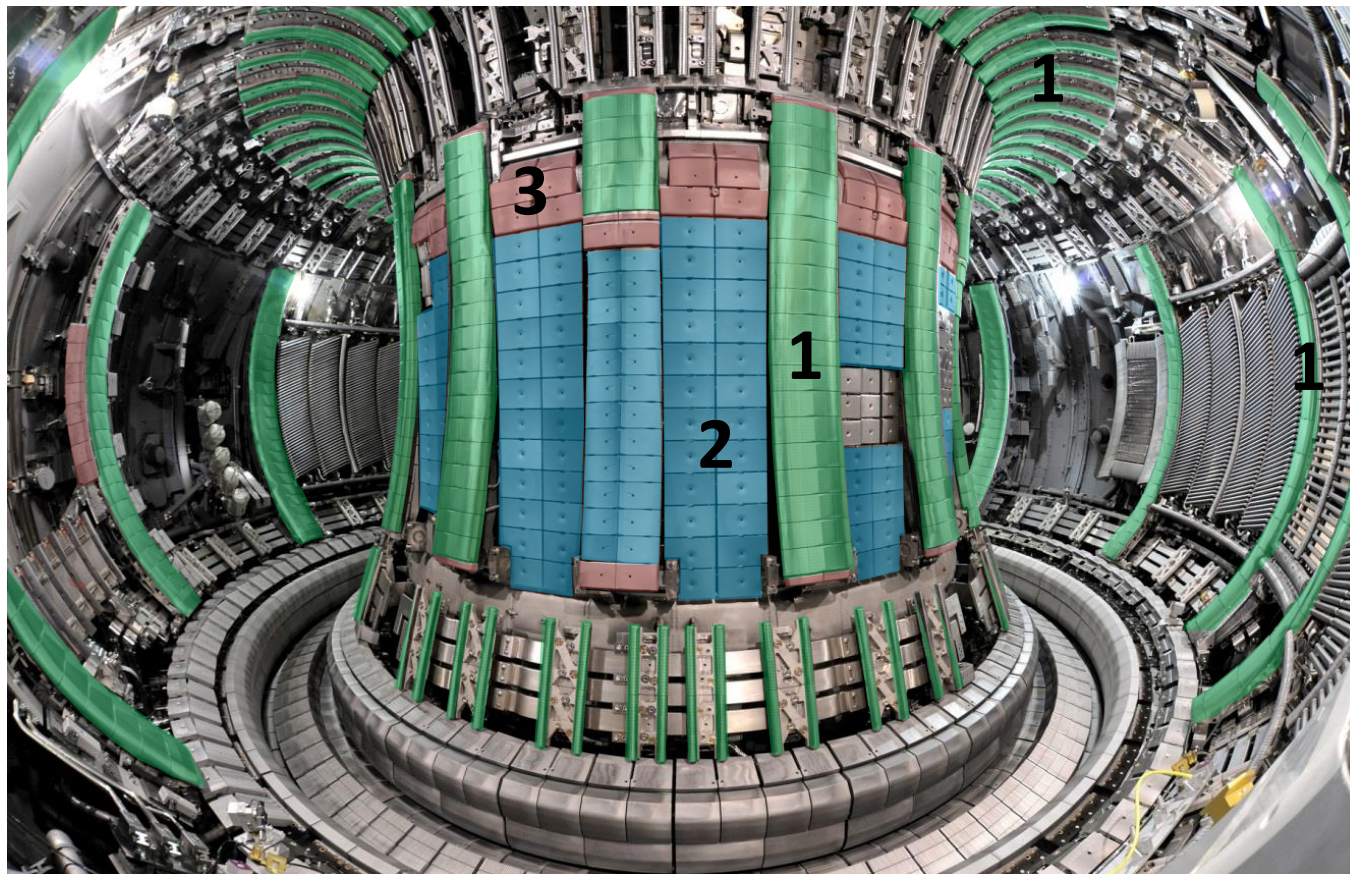
- 1** Bulk Be
- 2** Be coated inconel
- 3** W – coated CFC

DIVERTOR

- 3** W – coated CFC
- 4** Bulk W



Overview – JET's main chamber PFCs

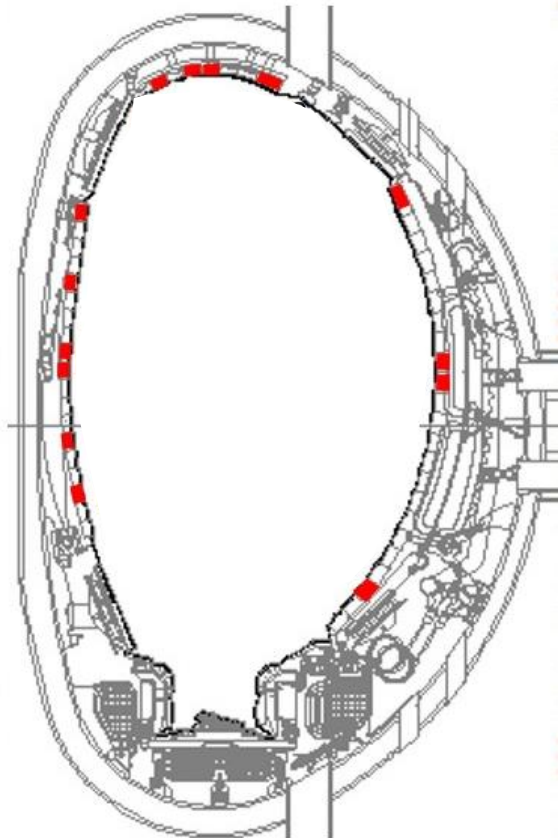


Main chamber

- 1** Bulk Be
- 2** Be coated inconel
- 3** W – coated CFC

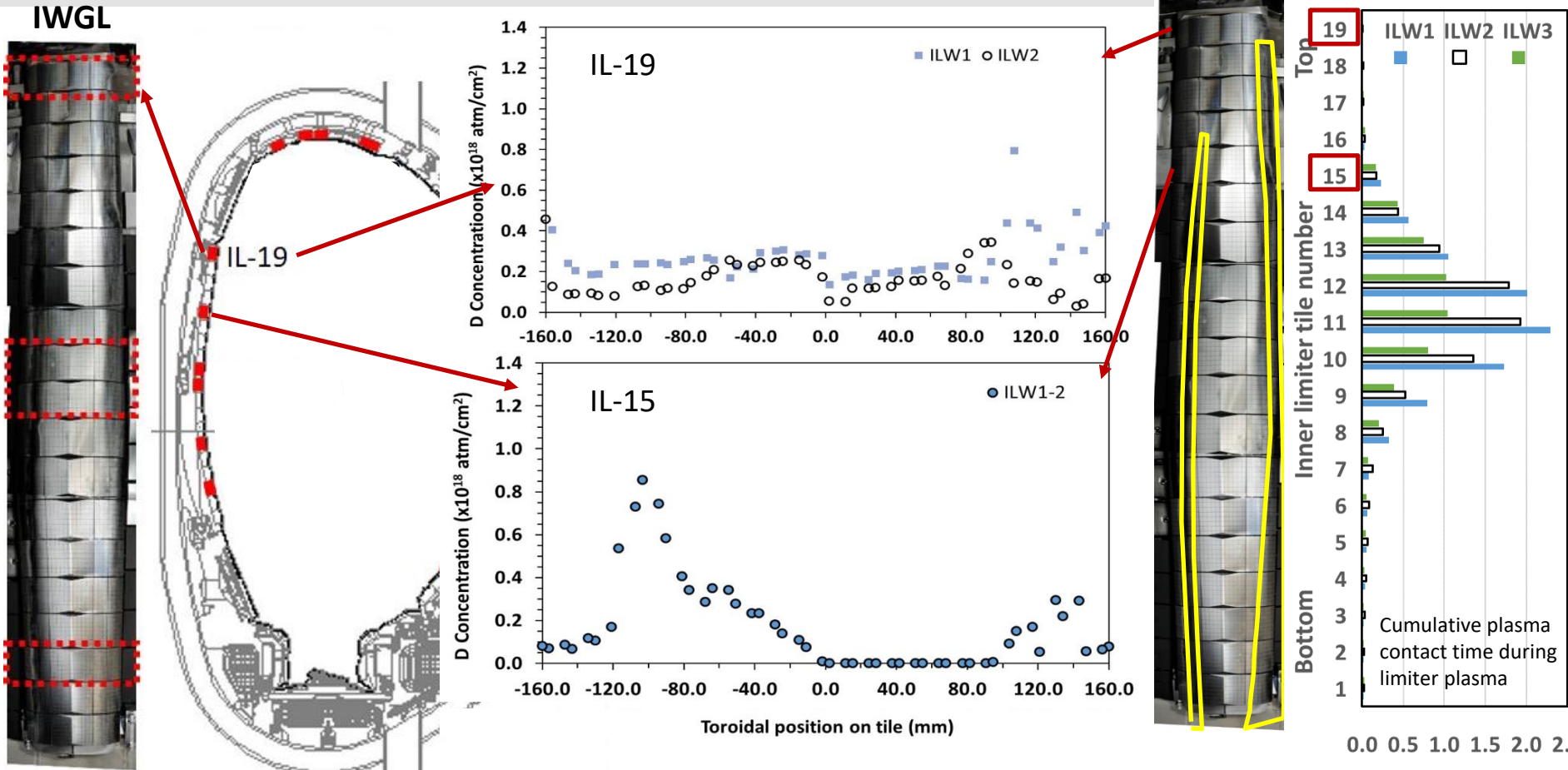
DIVERTOR

- 3** W – coated CFC
- 4** Bulk W





Overview – JET’s main chamber PFCs: Beryllium limiters – Fuel retention (IBA data)

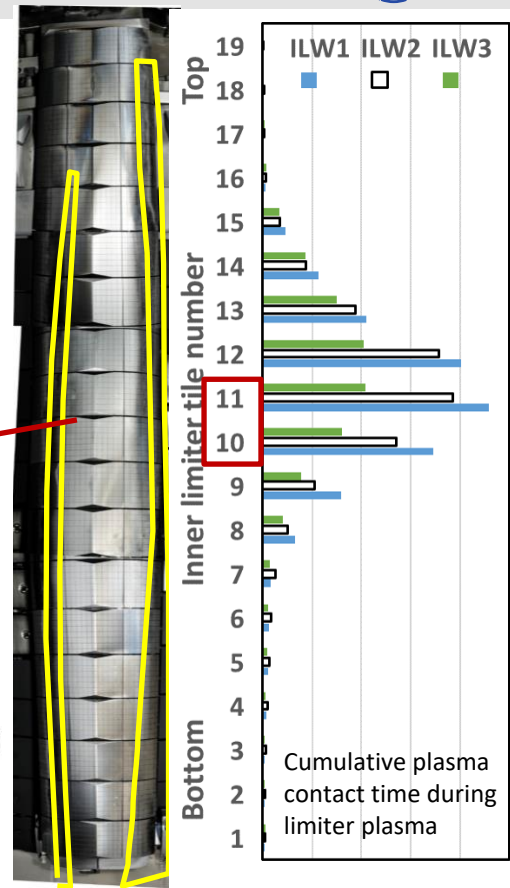
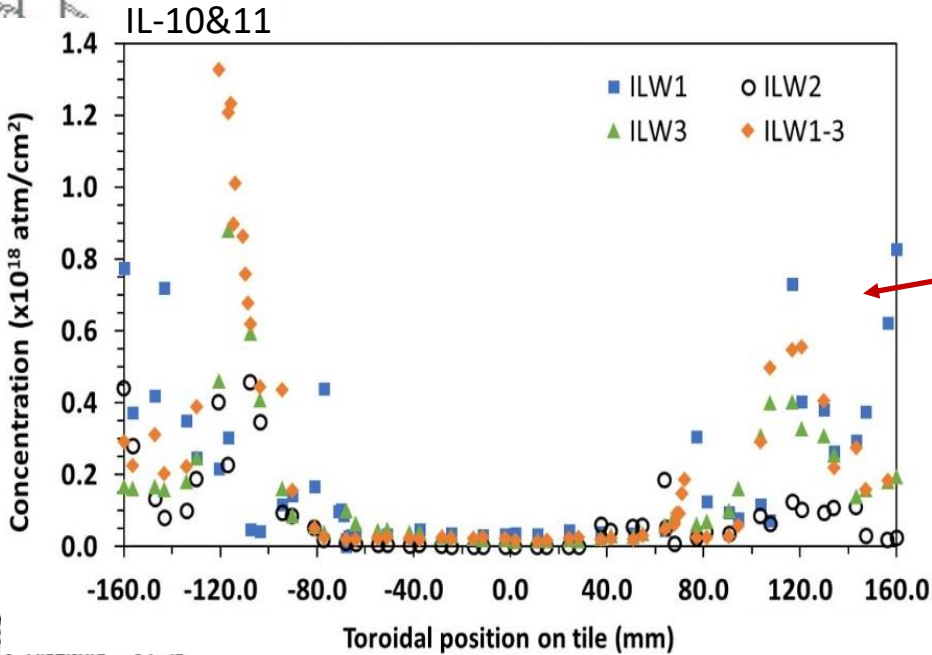
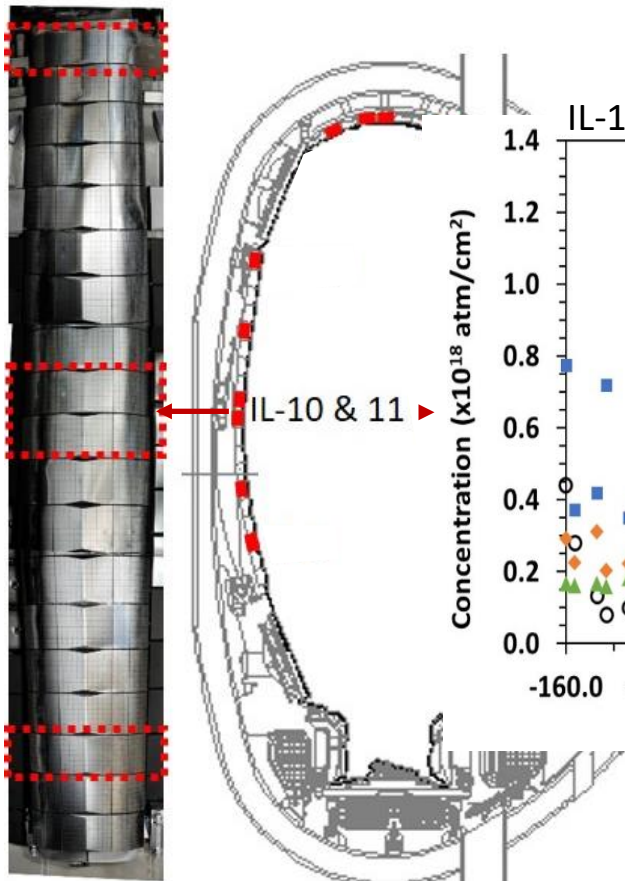


[1] A Widdowson et al 2020 Phys. Scr. 2020 014051



Overview – JET’s main chamber PFCs: Beryllium limiters – Fuel retention (IBA data)

IWGL

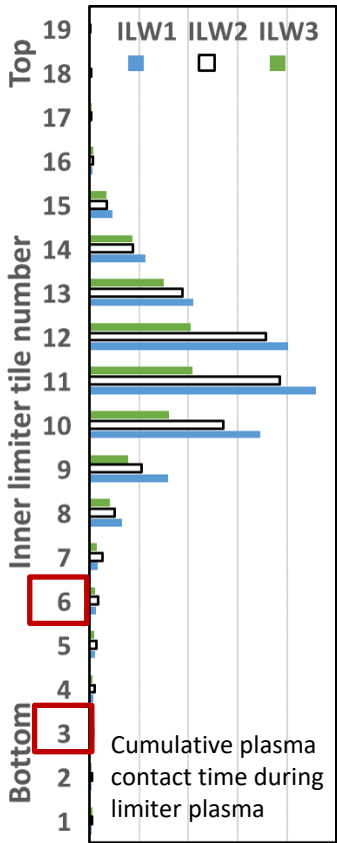
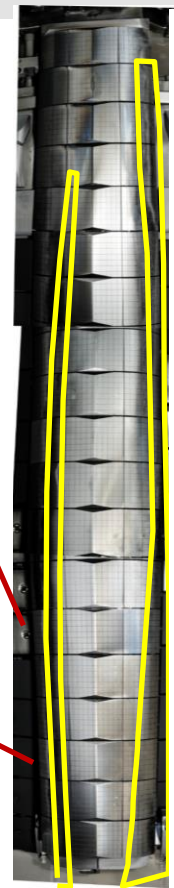
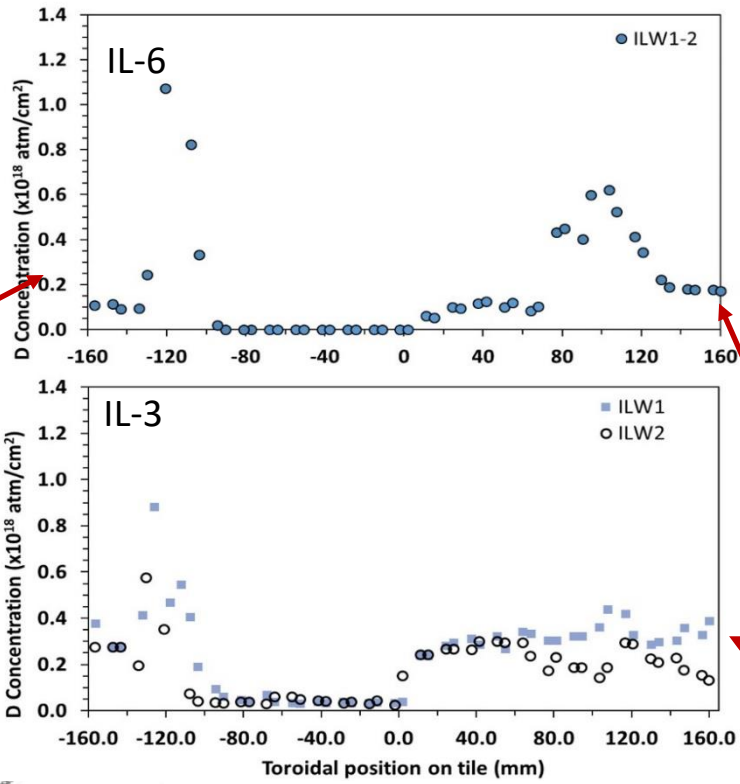
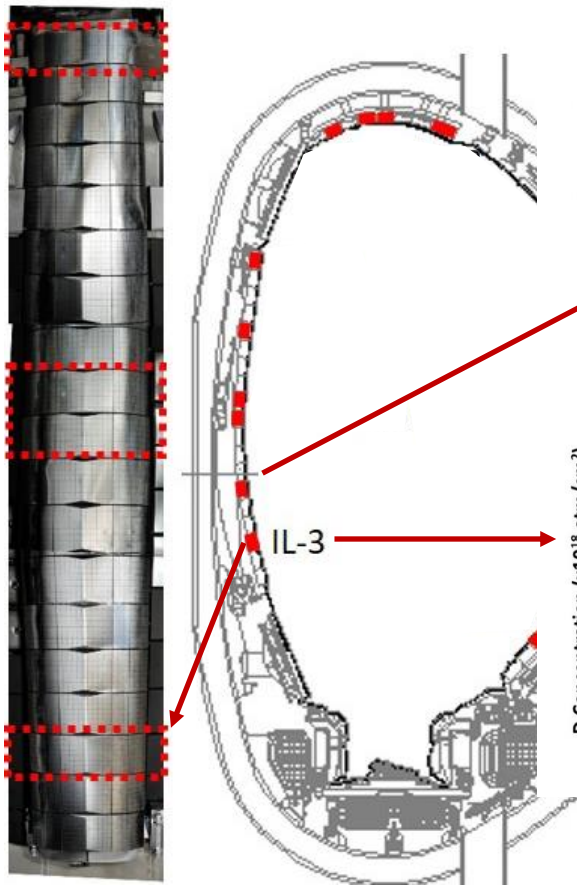


[1] A Widdowson et al 2020 Phys. Scr. 2020 014051



Overview – JET’s main chamber PFCs: Beryllium limiters – Fuel retention (IBA data)

IWGL



Hours

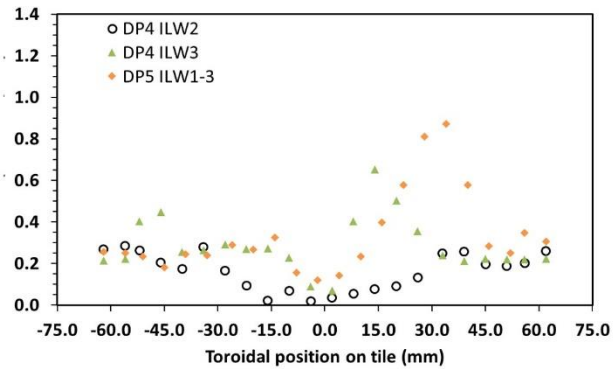
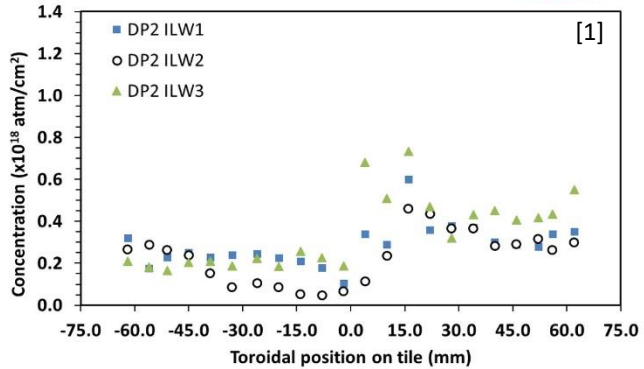
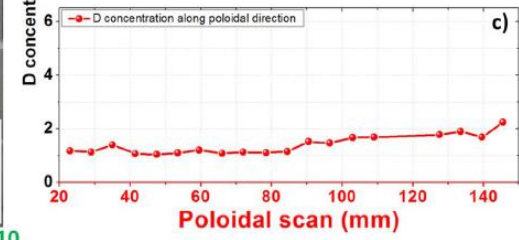
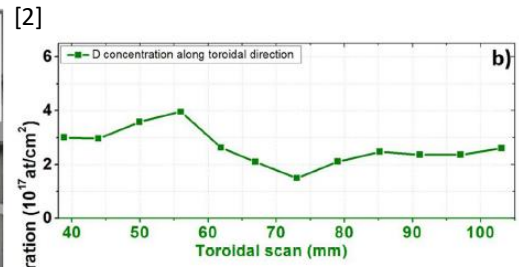
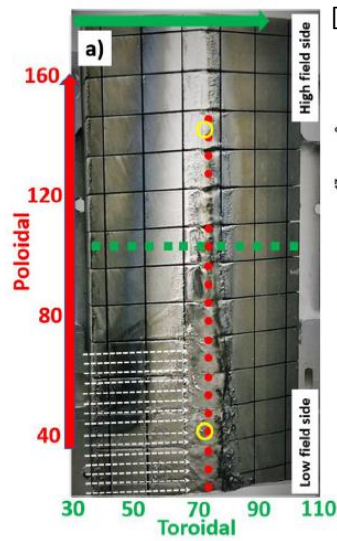
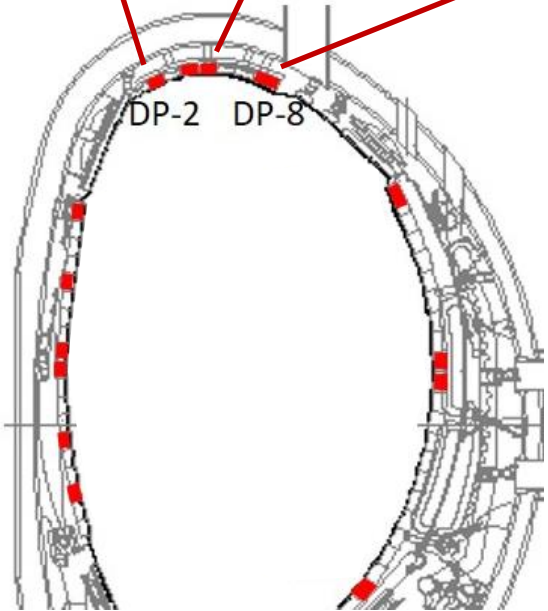
[1] A Widdowson et al 2020 Phys. Scr. 2020 014051



Overview – JET’s main chamber PFCs: Beryllium limiters – Fuel retention (IBA data)



DP

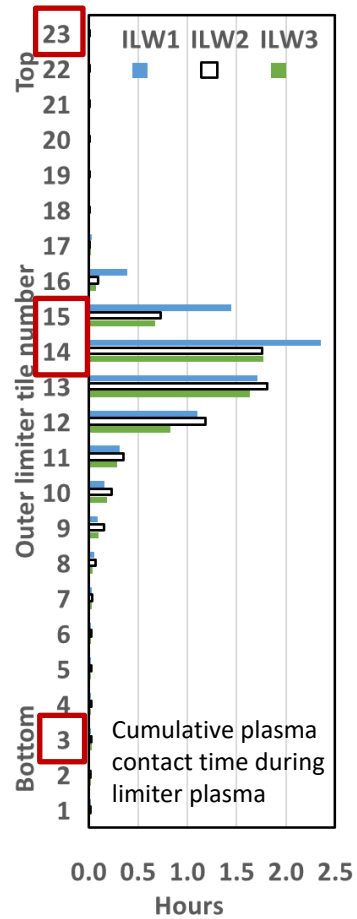
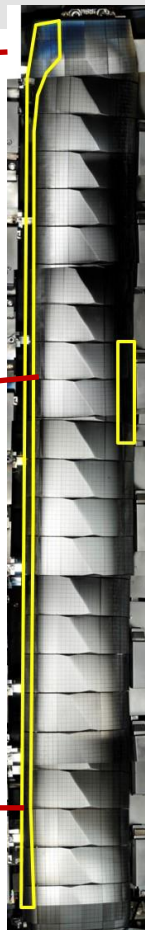
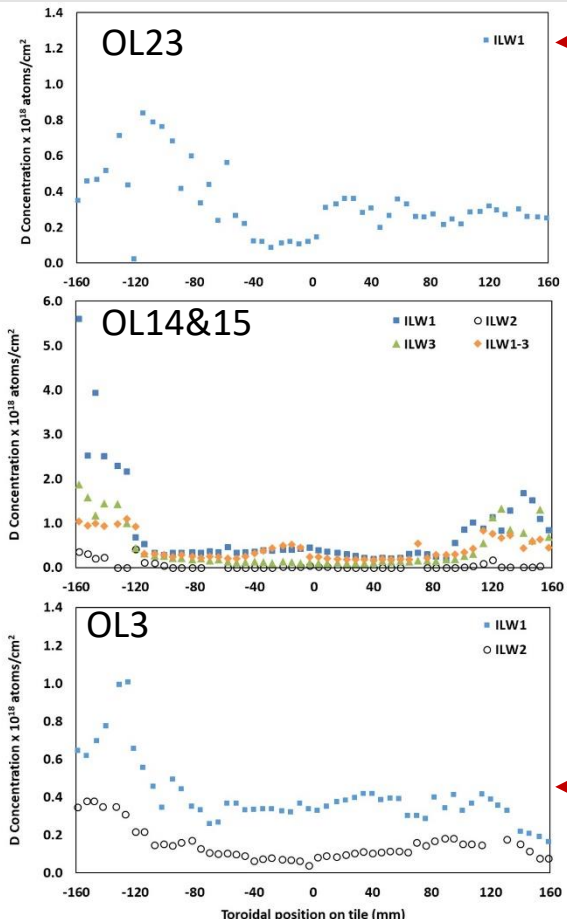
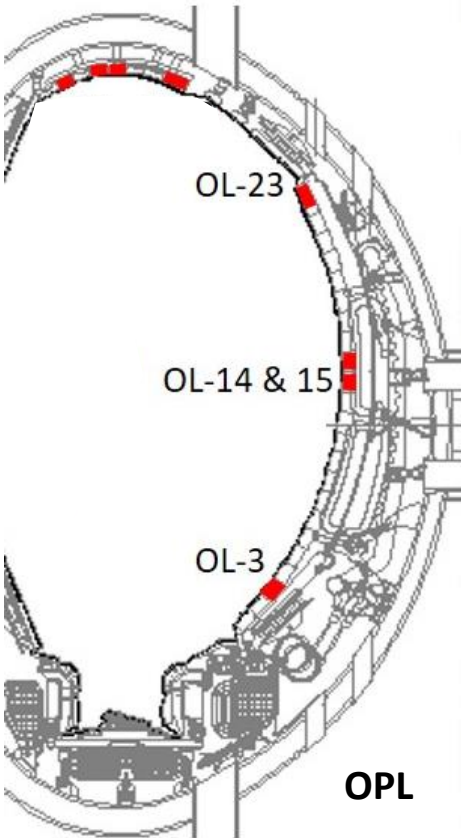


[1] A Widdowson et al 2020 Phys. Scr. 2020 014051

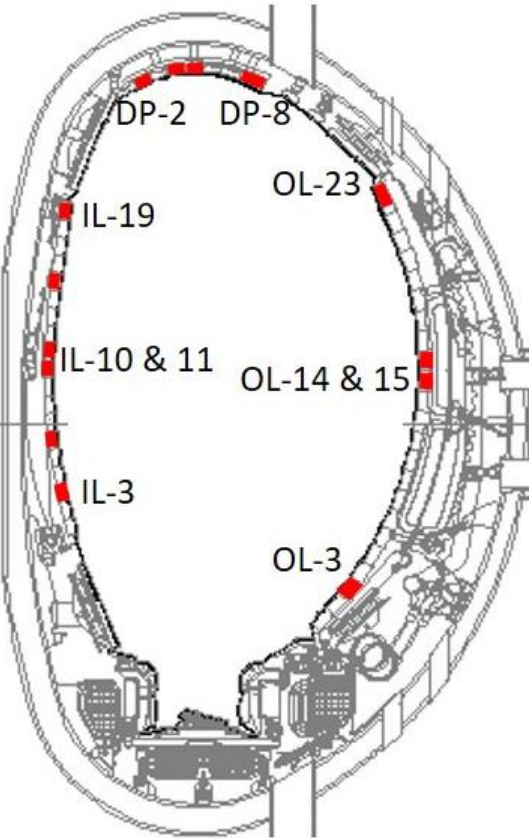
[2] I. Jezu et al 2019 Nucl. Fusion 59 086009



Overview – JET’s main chamber PFCs: Beryllium limiters – Fuel retention (IBA data)



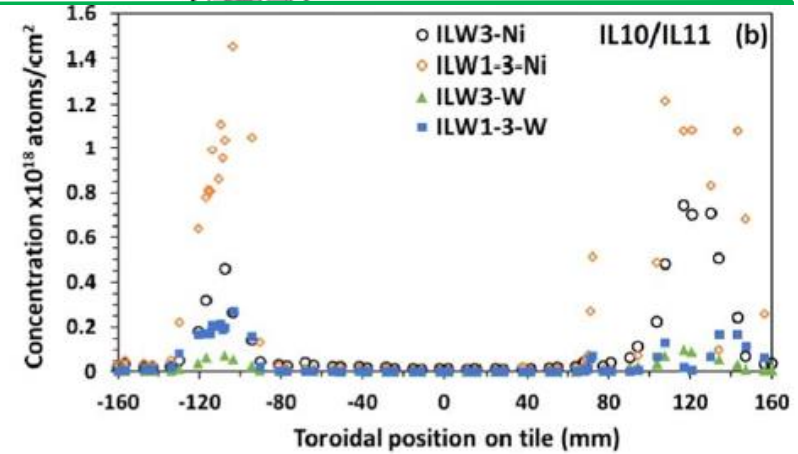
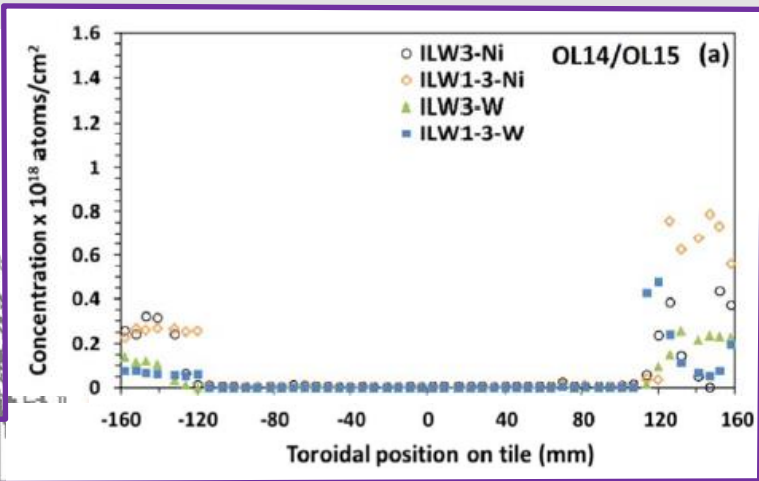
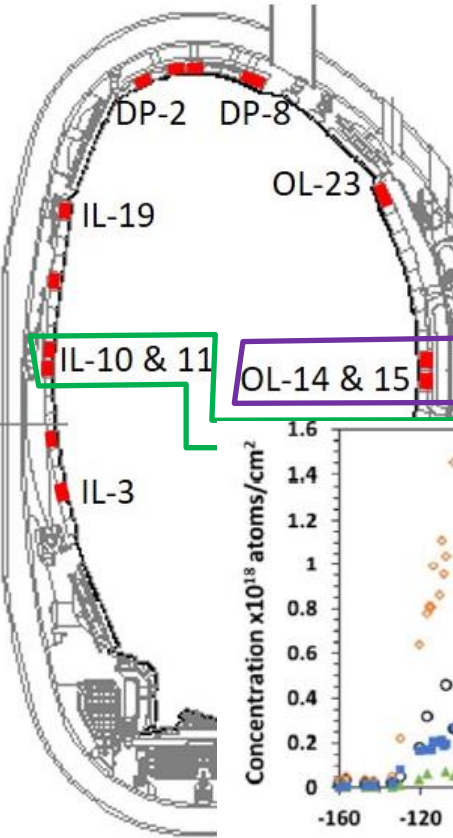
Overview – JET’s main chamber PFCs: Beryllium limiters – Fuel retention summary(IBA data)



Exposure period in JET of analysed tiles			
Inner limiter tiles	ILW1 (2011-12)	ILW2 (2013-14)	ILW3 (2015-16)
IL-3	←→	←→	←→
IL-6	No IBA data		←→
IL-10	←→	←→	←→
IL-11	←→		
IL-15	No IBA data		←→
IL-19	←→	←→	←→
Outer limiter tiles			
OL-3	←→	←→	←→
OL-14	←→	←→	←→
OL-15	←→		
OL-23	←→	No IBA data	No IBA data
Dump plate tiles			
DP-2	←→	←→	←→
DP-4	←→	←→	←→
DP-5	←→		
DP-8	←→		

- Fuel retention summary**
- Fuel retention on limiter tiles concentrated in deposits
 - Contribution to total fuel retention in surface of beryllium limiter tiles <0.03%
 - Retention lowest in high heat load/erosion areas at midplane inner/outer limiters & dump plate ridge
 - Deposits for ILW1-3 tiles thicker than analysis depth therefore total deuterium underestimated

Overview – JET’s main chamber PFCs: Beryllium limiters – Material migration (Mid&high-Z)



Deposit
Beryllium 7 μm
Nickel 3μm
Beryllium tile

Material Migration

- Erosion of inner and outer beryllium limiters occurs mainly during limiter phase;
- For all three ILW operating periods erosion/deposition pattern shows similar distribution
- Tungsten, nickel, iron, chromium found in deposits for all ILW operating periods

Mid-Z and high-Z impurities migrate throughout vessel

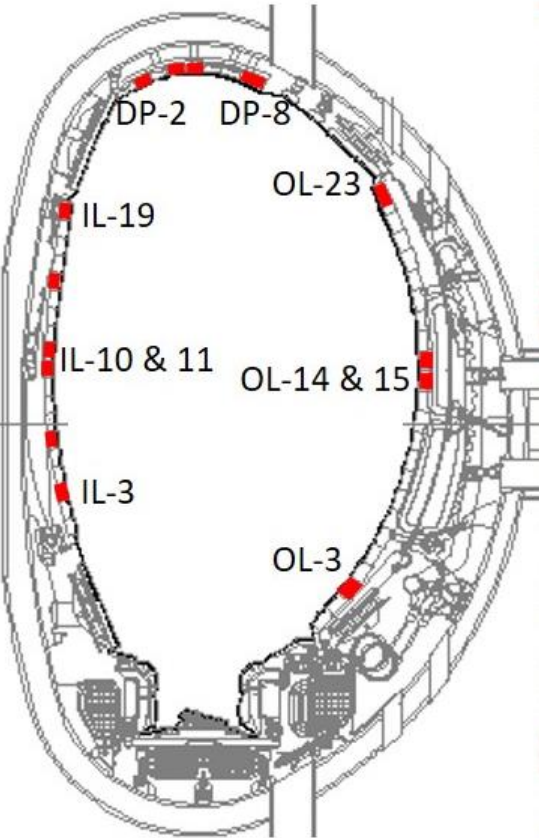
- Roughly follow global deposition patterns for beryllium
- Impurities migrate until they reach a remote region not accessible to plasma**
- Upper inner divertor, divertor corners and ends of main chamber limiter tiles

Campaign to campaign variations in concentration are modest

- Deposition patterns influenced by plasma configurations and operating conditions

Continuing sources of W, Ni, Cr, Fe throughout all operating periods

Ratios of Ni, Cr, Fe impurities show that Inconel is the source

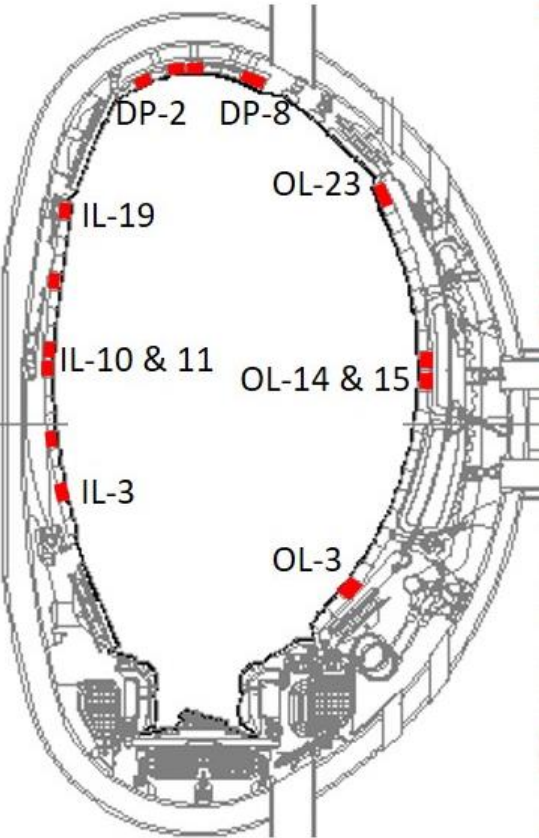


Deposit
Beryllium 7 μm
Nickel 3 μm
Beryllium tile





Overview – JET’s main chamber PFCs: Beryllium limiters – Material migration (Low-Z)



Deposit
Beryllium 7 μm
Nickel 3μm
Beryllium tile



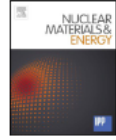
Nuclear Materials and Energy 18 (2019) 331–338



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journal homepage: www.elsevier.com/locate/nme



Beryllium global erosion and deposition at JET-ILW simulated with ERO2.0

J. Romazanov^{a,a}, S. Brezinsek^a, D. Borodin^{1,a}, M. Groth^b, S. Wiesen^a, A. Kirschner^a, A. Huber^a, A. Widdowson^c, M. Airila^d, A. Eksaeva^{a,c}, I. Borodkina^{a,c}, Ch. Linsmeier^a, JET Contributors²

^a Forschungszentrum Jülich GmbH, Institut für Energie- und Klimaforschung – Plasmaphysik, Partner of the Trilateral Eurolog Cluster (TEC), Jülich 52425, Germany
^b Association EURATOM-Tekes, Aalto University, Espoo, Finland
^c Culham Centre for Fusion Energy, Abingdon OX14 3DB, UK
^d VTT Technical Research Centre of Finland Ltd., PO Box 1000, FI-02044 VTT, Finland
² National Research Nuclear University MEPhI, 31, Kashirskoe sh., Moscow 115409, Russia





Overview – JET’s main chamber PFCs: Beryllium limiters – VDEs damage

Plasma disruption

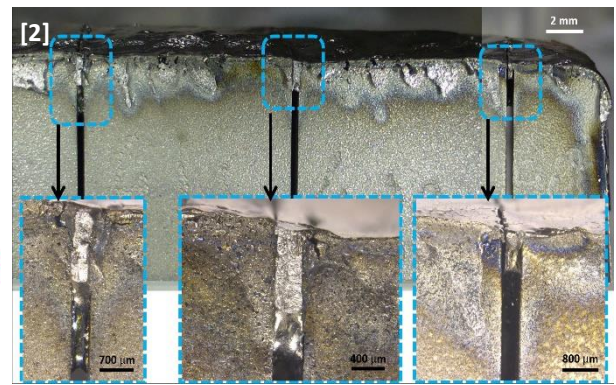
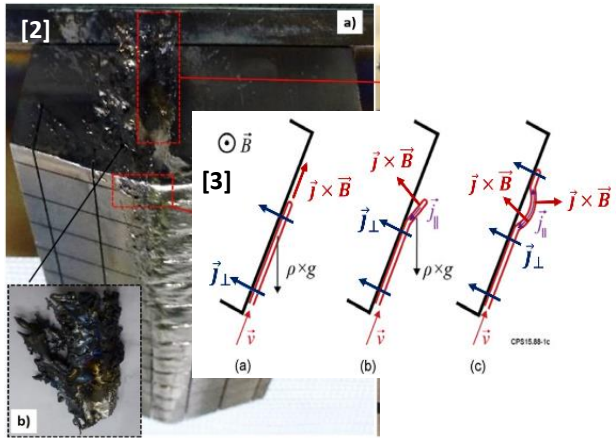
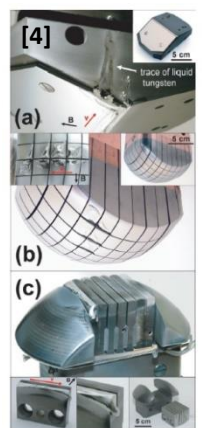
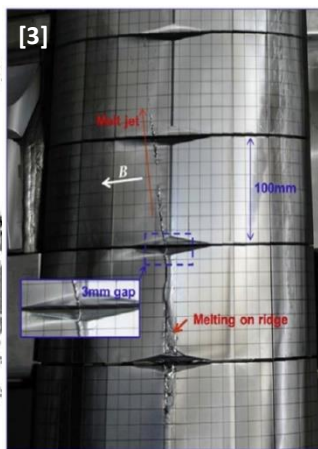
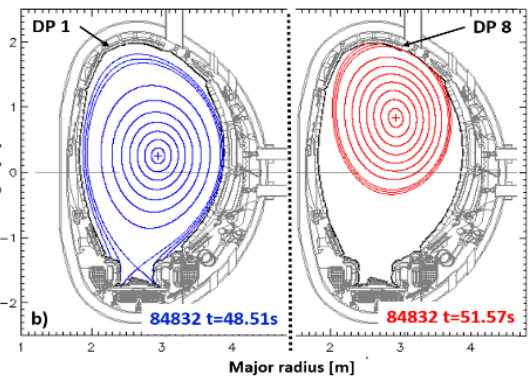
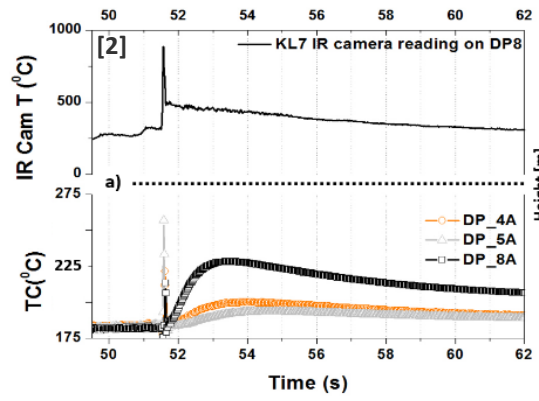
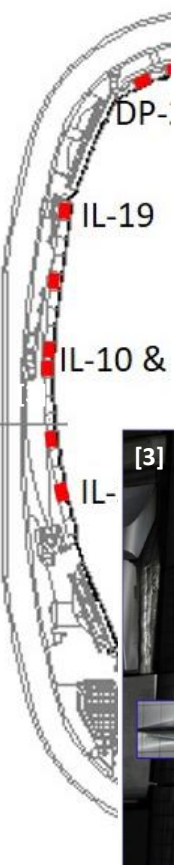
Termination of plasma with rapid loss of thermal and magnetic energy



HUGE thermal and mechanical loads on the structure

Consequences of the disruptions

- Thermal loads/Fast melting and electromagnetic forces



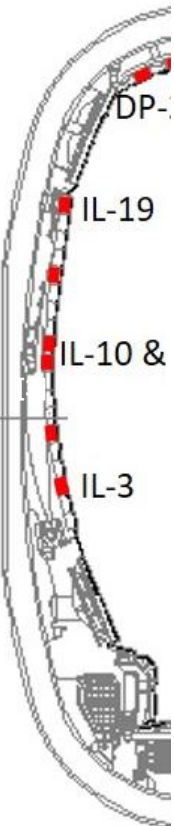
[2] I. Jecu et al, Nucl. Fusion 59 (2019) 086009
 [3] G.F. Matthews, et al., Phys. Scr. T167 (2016) 014070 (7pp)
 [4] G. Sergienko et al., Phys. Scr. T128 (2007) 81–86



Overview – JET’s main chamber PFCs: Beryllium limiters – VDEs damage - modelling

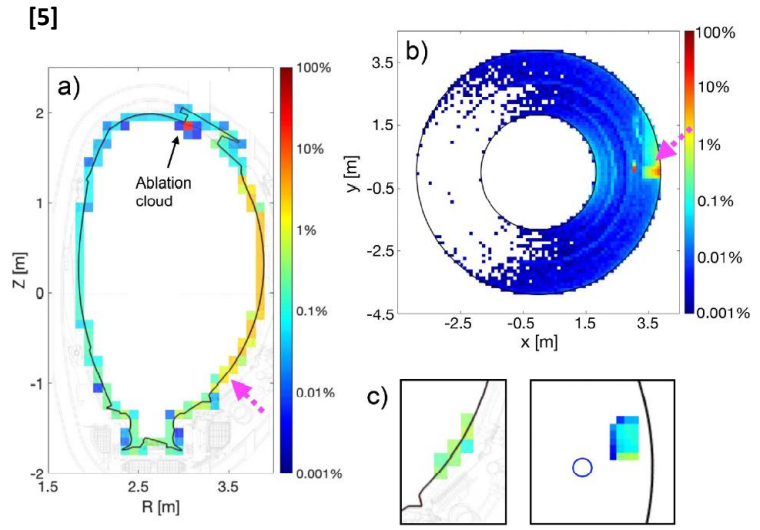
IOP Publishing Plasma Physics and Controlled Fusion
Plasma Phys. Control. Fusion 62 (2020) 064001 (11pp) <https://doi.org/10.1088/1361-6587/ab8610>

OPEN ACCESS IOP Publishing | International Atomic Energy Agency Nuclear Fusion
Nucl. Fusion 62 (2022) 036016 (10pp) <https://doi.org/10.1088/1741-4326/ac47b7>



An insight on beryllium dust sources in the JET ITER-like wall based on numerical simulations

Plasma Phys. Control. Fusion 62 (2020) 064001 A Uccello et al



A. Uccello et al, Plasma Phys. Control. Fusion 62 (2020) 064001 (11pp)

[5] A. Uccello et al, Plasma Phys. Control. Fusion 62 (2020) 064001 (11pp)

Simulations of liquid metal flows over plasma-facing component edges and application to beryllium melt events in JET

L. Vignitchouk et al. Nucl. Fusion 62 (2022) 036016 (10pp)

Nucl. Fusion 62 (2022) 036016 L. Vignitchouk et al

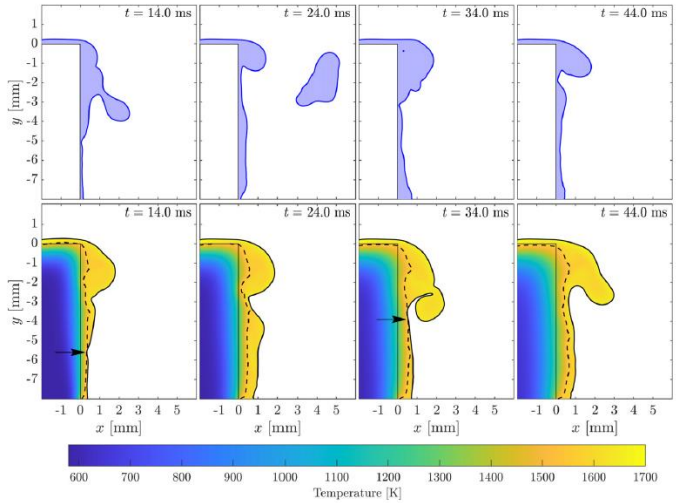


Figure 7. Comparison of simulation results with (lower row) and without (upper row) heat transfer for $h = 200 \mu\text{m}$ and $v = 3 \text{ m s}^{-1}$. Dashed black curves in the lower row highlight the position of the re-solidification front. Arrows indicate locations where the free surface and the re-solidification front are found to meet tangentially, meaning that the liquid domain is split in two nearly independent layers. The older layer is almost fully solidified and behaves effectively as a substrate on which the newer layer propagates. Video clips are provided as supplementary material.

[5] A. Uccello et al, Plasma Phys. Control. Fusion 62 (2020) 064001 (11pp)

[6] L. Vignitchouk et al. Nucl. Fusion 62 (2022) 036016 (10pp)

Overview – JET’s main chamber PFCs: Beryllium limiters – VDEs damage - modelling

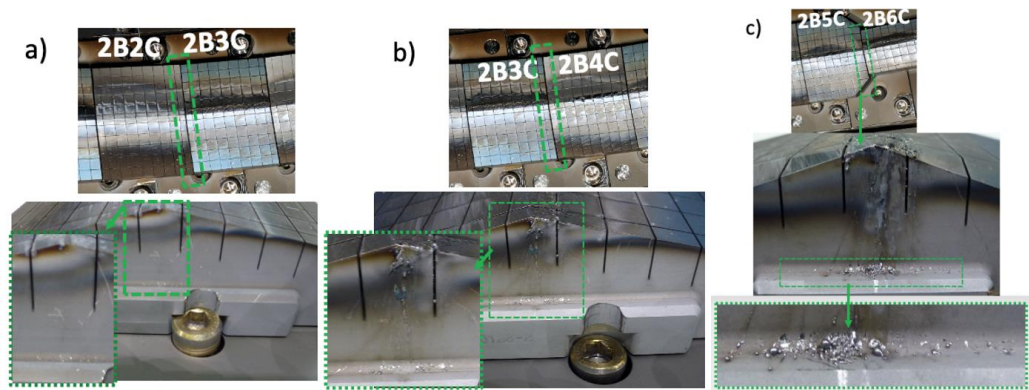


Figure S5 a) Close-up of Be droplets found between DP-2 and DP-3; b) close-up of Be droplets between DP-3 and DP-4 and c) close-up of Be droplets found in gaps between tiles DP-5 and DP-6.

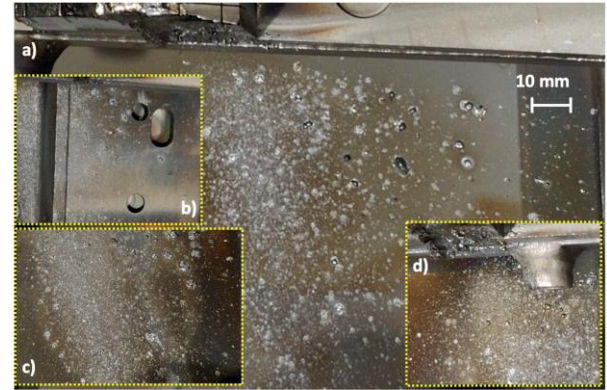
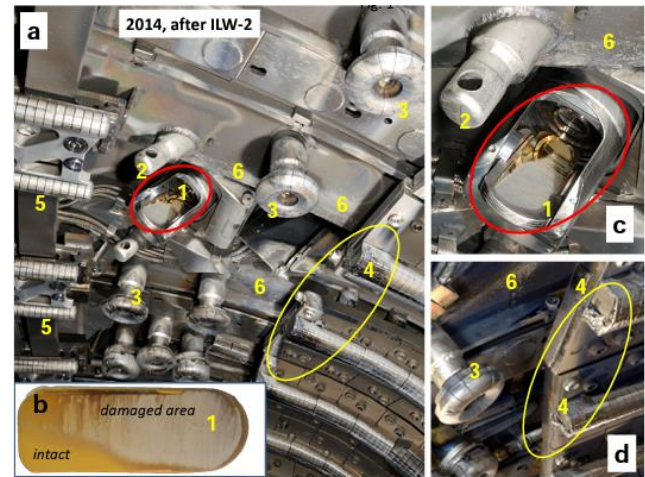
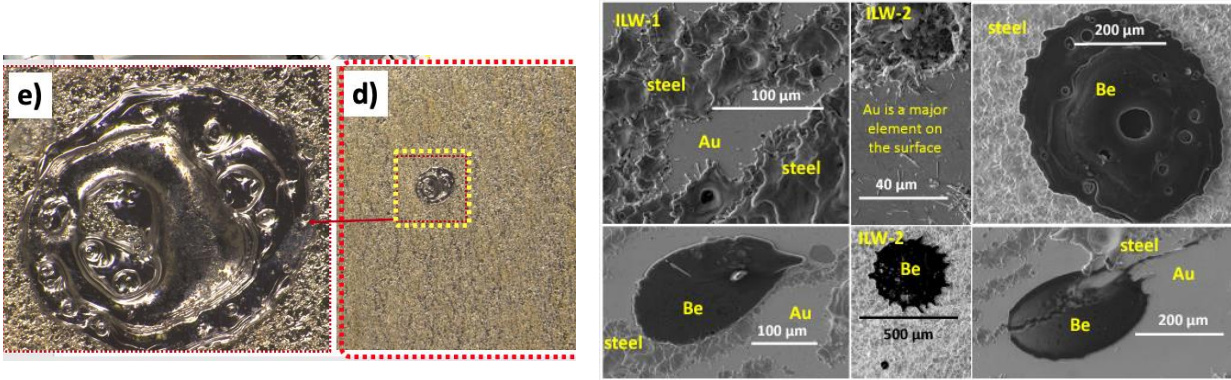


Figure S7 Close-up of Be droplets found on the outer wall in the vicinity of DP-8 with a) a scale dimension for a better understanding of the size of droplets found on the walls, while b), c) and d) are examples of droplets in different regions of the upper vessel. Scale in a) does not apply in b), c) and d).





Overview – JET’s main chamber PFCs: Beryllium limiters – REs damage

Plasma disruption

Termination of plasma with rapid loss of thermal and magnetic energy



HUGE thermal and mechanical loads on the structure

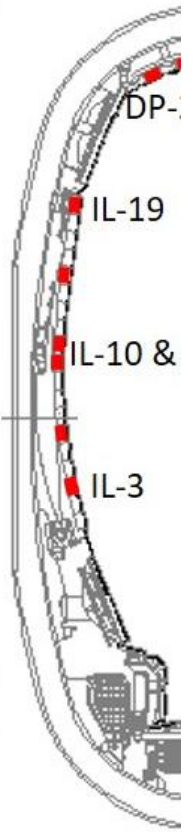
Consequences of the disruptions

- Thermal loads/Fast melting and electromagnetic forces

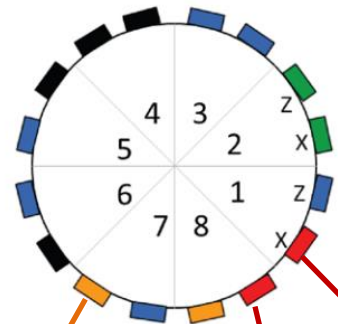
- **High energy Runaway Electrons (RE)**



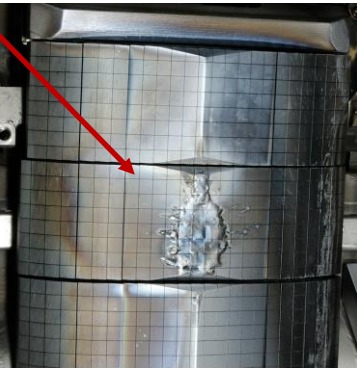
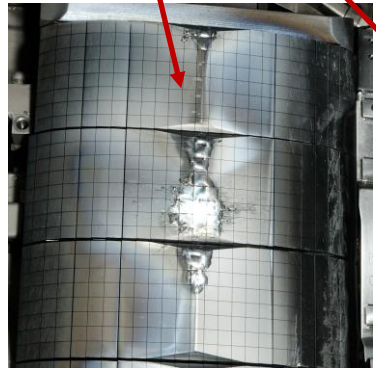
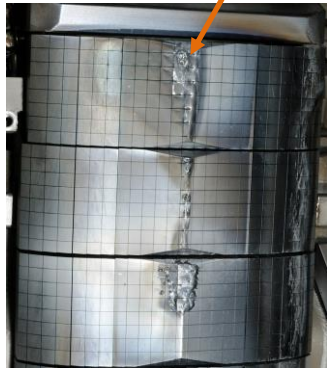
Serious threats to future tokamaks



[7]



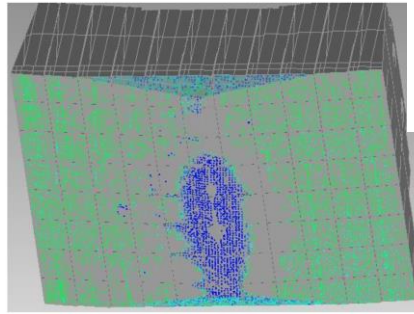
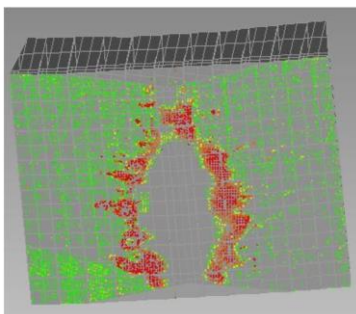
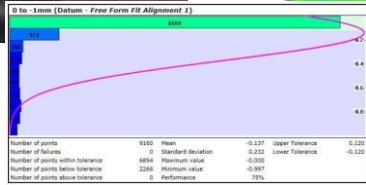
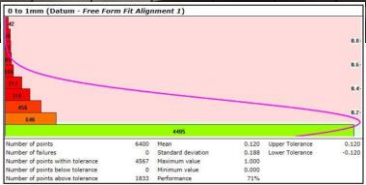
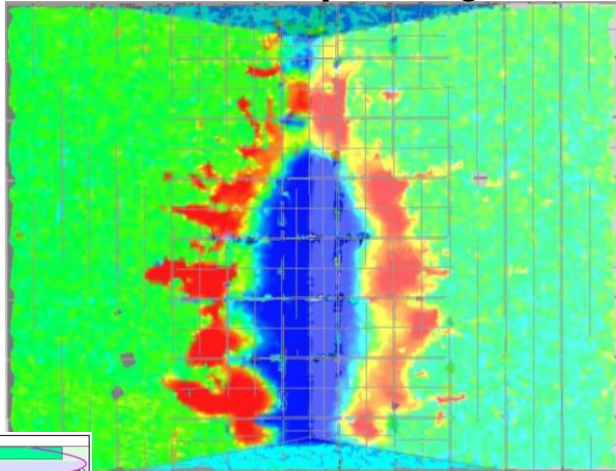
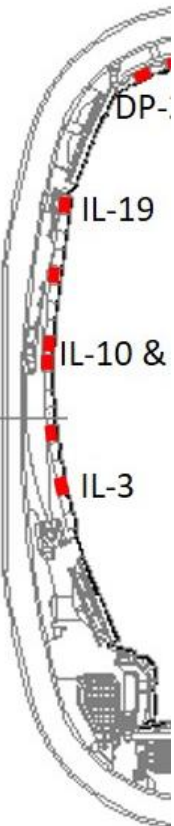
Red	Significant melting
Yellow	Traces of melting
Green	Surface alteration only
Blue	No damage
Black	No data



[7] C. Reux et al Nucl. Fusion 55 (2015) 093013



Overview – JET’s main chamber PFCs: Beryllium limiters – REs damage Laser 3D profiling

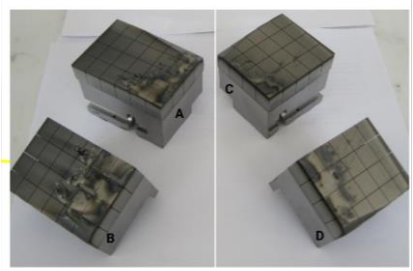
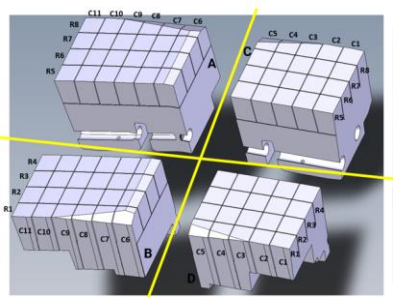
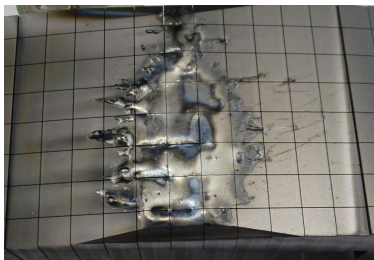


Modelling of the RE induced damage to PFC
L. Chen, R.A. Pits, M. Lehnen, C. Reux, S. Ratynskaia et al.





Overview – JET’s main chamber PFCs: Beryllium limiters – REs damage

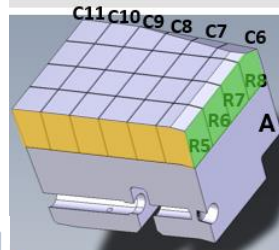
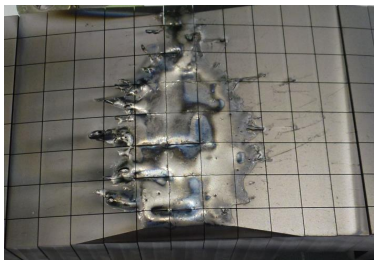
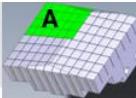




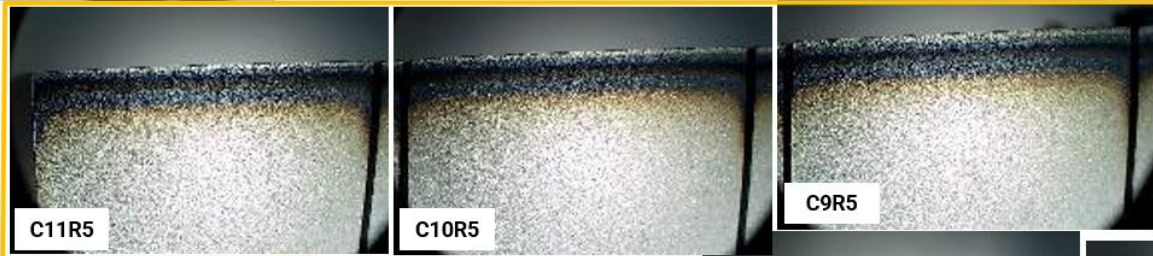
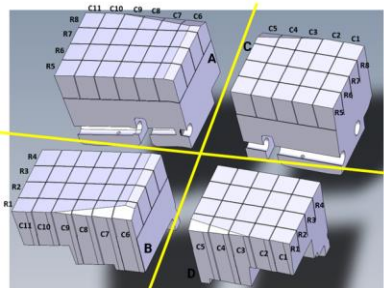
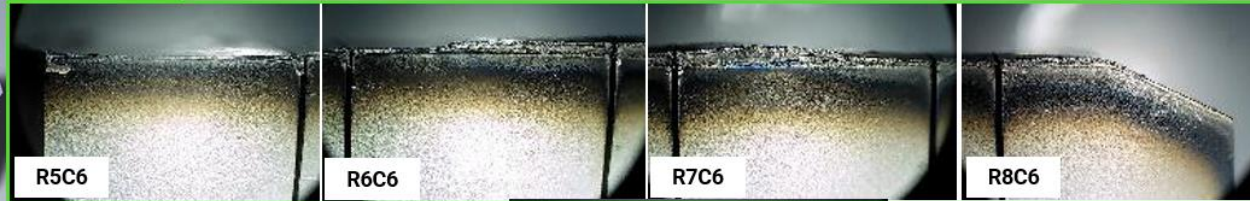
Overview – JET’s main chamber PFCs: Beryllium limiters – REs damage

REs damage to JET PFC – Be limiter tile

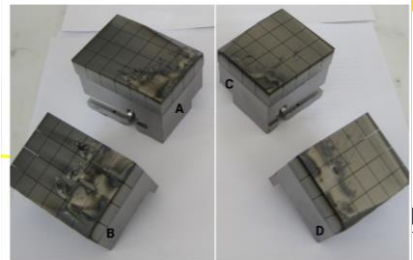
*Imaging and Microscopy



Poloidal direction



Toroidal direction

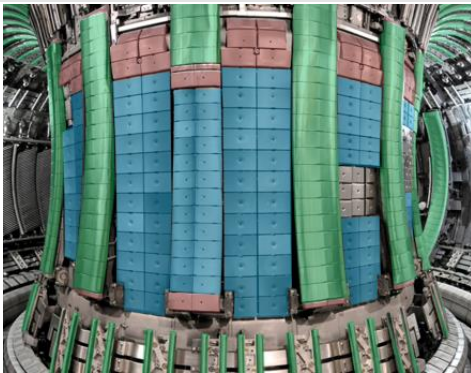


performed at IAP

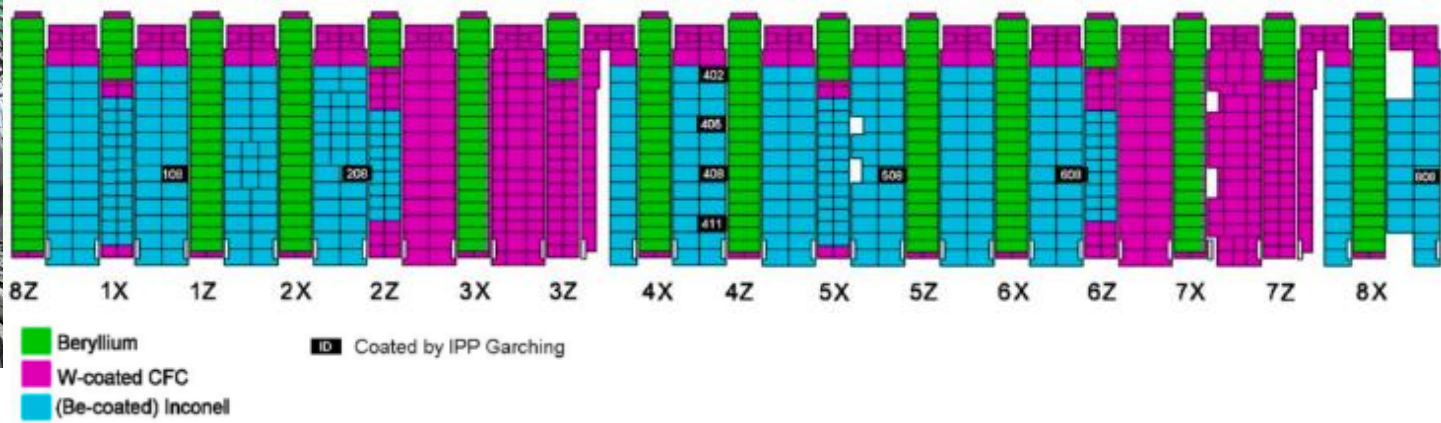
I. Jepu | RE impact on JET PFC | SPE meeting | Zoom | 20.10.2022



Overview – JET’s main chamber PFCs: Recessed inner wall



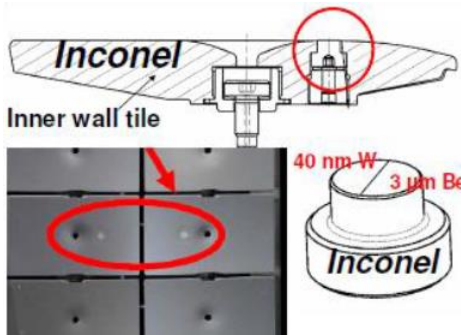
Be coated inconel



[8] S. Krat et al. Nuclear Materials and Energy 29 (2021) 101072



Overview – JET’s main chamber PFCs: Recessed inner wall – Be/W wall inserts

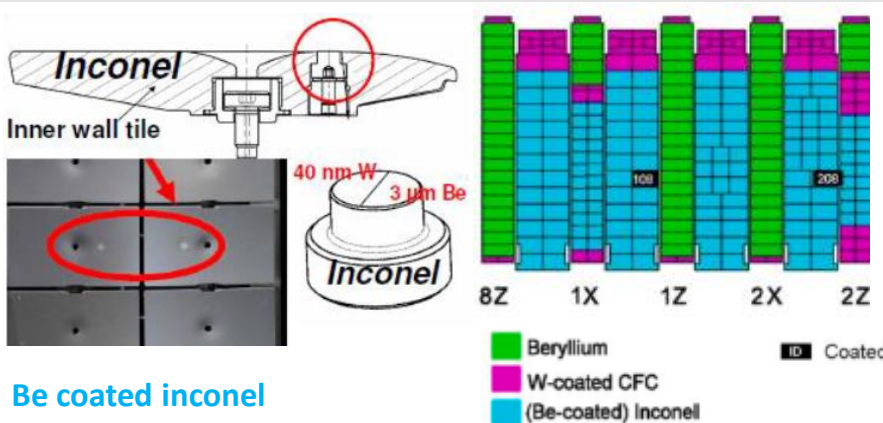


Be coated inconel

[8] S. Krat et al. Nuclear Materials and Energy 29 (2021) 101072



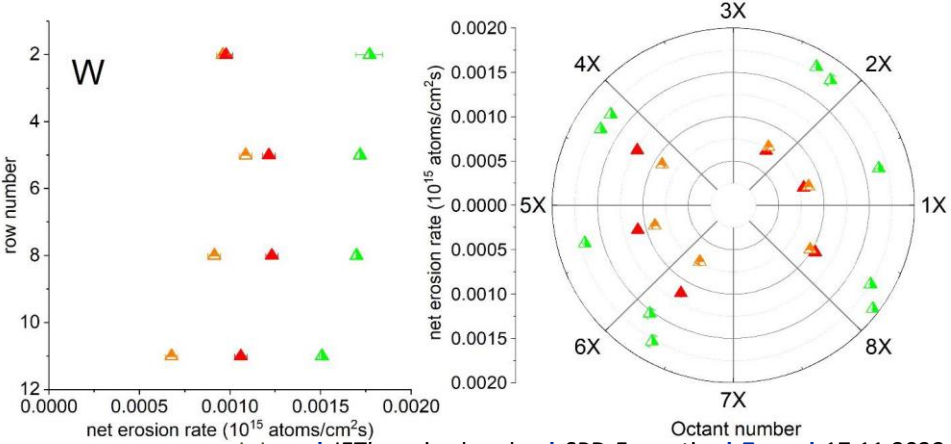
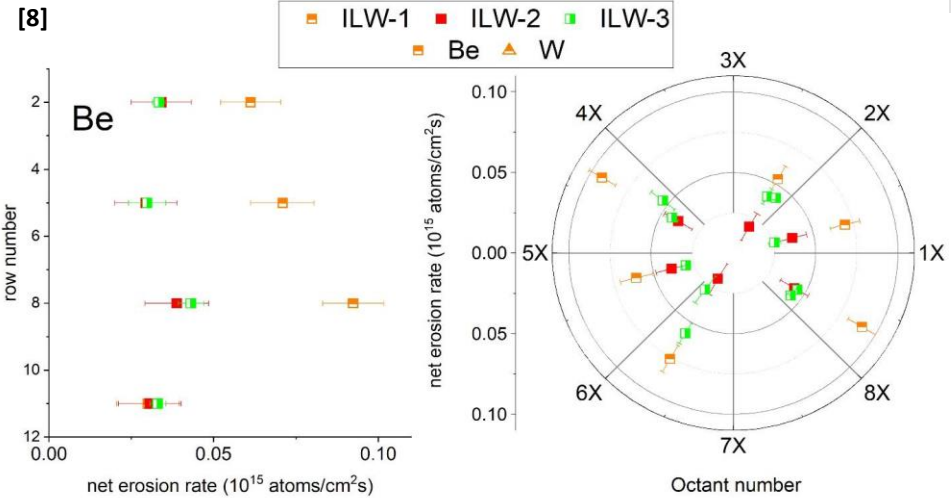
Overview – JET’s main chamber PFCs: Recessed inner wall – Be/W wall inserts



Be coated inconel

Results

- Fairly homogeneous erosion rate in toroidal and poloidal directions, both for Be & W.
- Variation of erosion rate within factor 2 for all 3 ILW campaigns
- Very small erosion rate for W < 0.3 pm/s



[8] S. Krat et al. Nuclear Materials and Energy 29 (2021) 101072

Overview – JET’s main chamber PFCs: Recessed inner wall – Be-coated Inconel (ERDA and $^3\text{He-NRA}$)

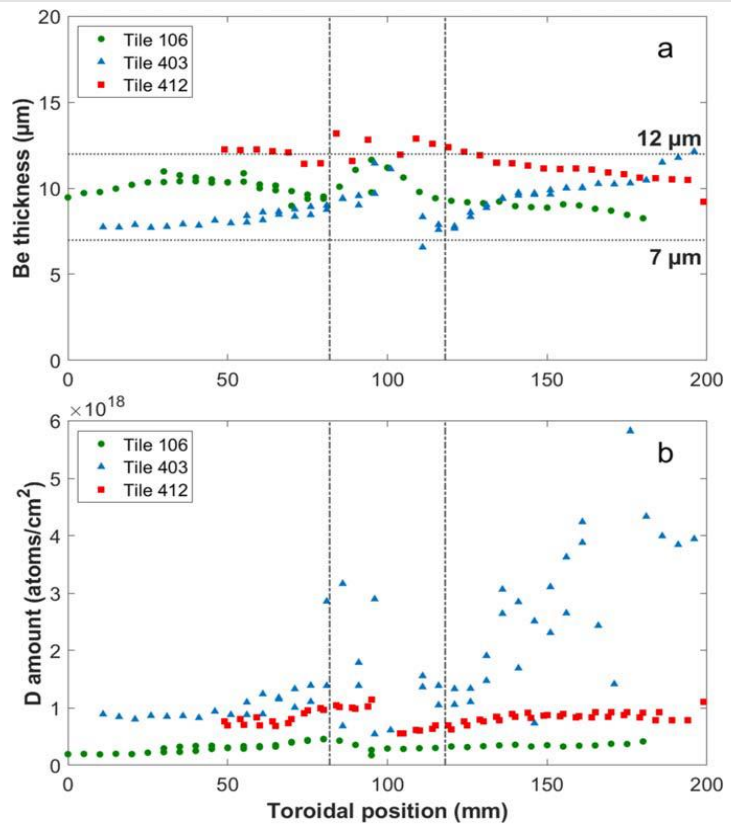
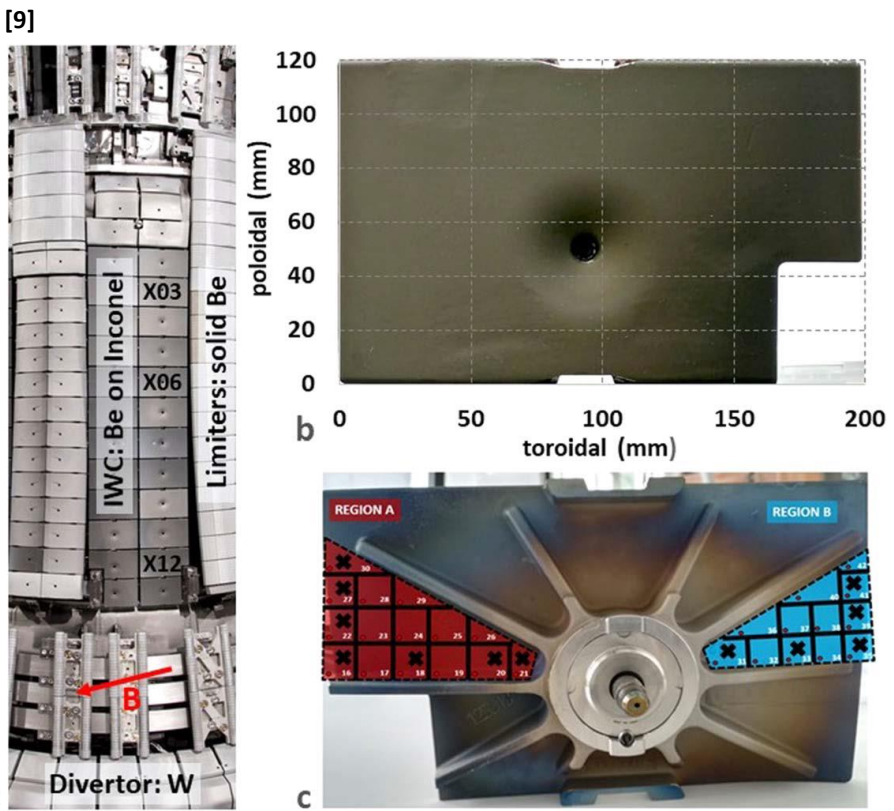
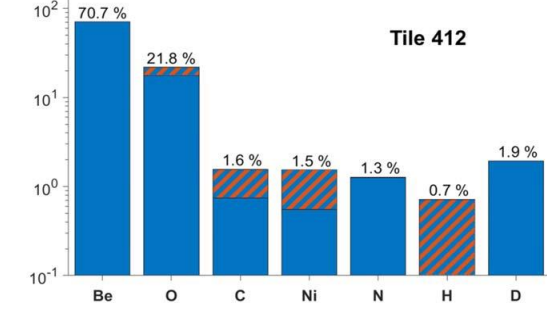
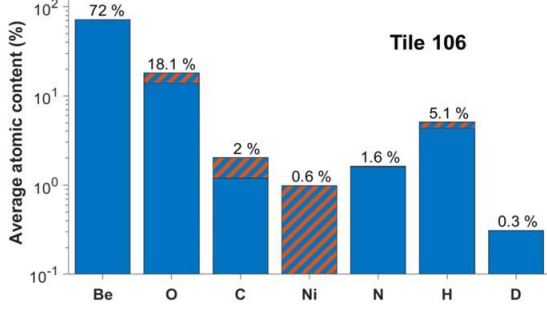
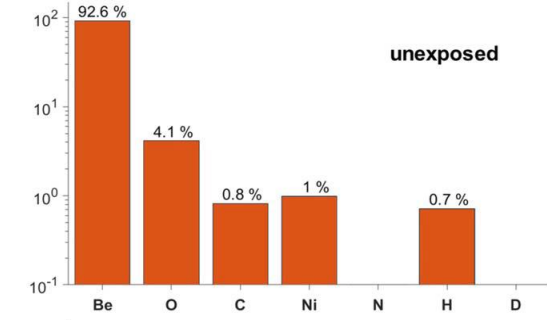
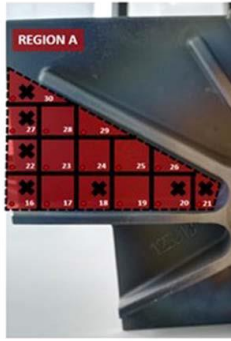
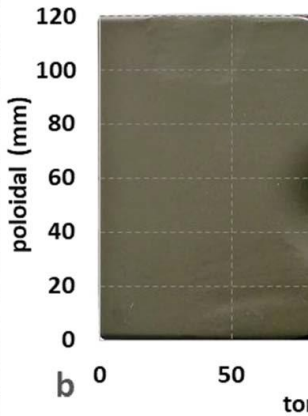
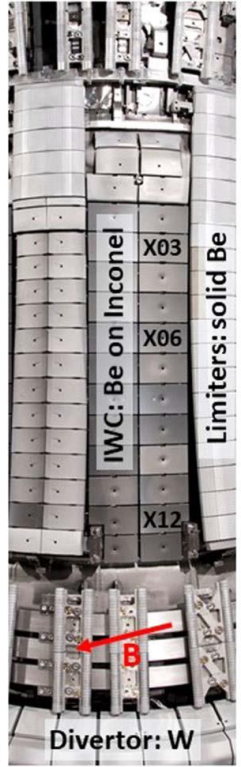


Figure 2. Toroidal profiles of (a) beryllium layer thickness and (b) deuterium retention on the IWC tiles, measured with a 2.3 MeV $^3\text{He}^+$ and H^+ beam. The Be layer thickness is given for an assumed material density of $1.85 \frac{\text{g}}{\text{cm}^3}$.



Overview – JET’s main chamber PFCs: Recessed inner wall – Be-coated Inconel (ERDA and ³He-NRA)

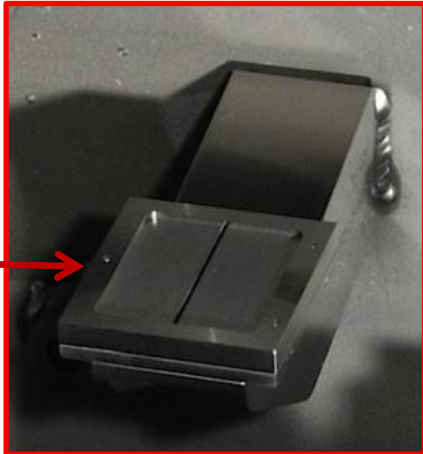
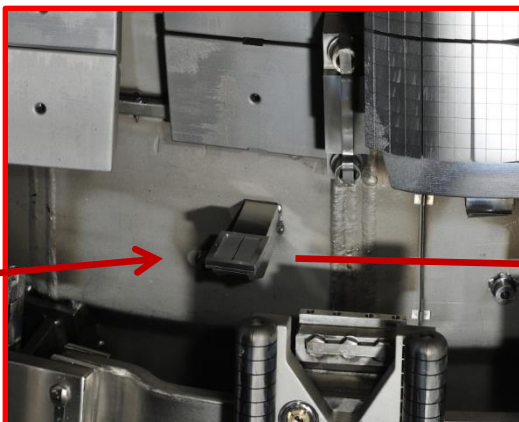
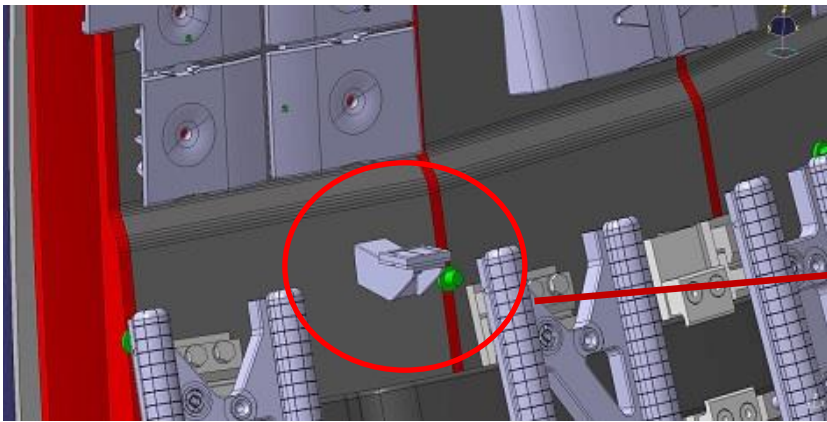
[9]



Results

- Total D retention amounts to 5.3×10^{22} Datoms (176mg) in the entire IWC beryllium coating for ILW 1–3 (equal to the sum of Datoms on IWGL and OPL, as assessed after ILW1–3 and it is five times smaller than determined for the divertor);
- The Be layer thickness on the exposed tiles is not uniform, but in neither place it is less than $7 \mu\text{m}$ thick;
- There are no signs of complete erosion. This proves good long-term adherence to the Inconel substrate;
- No material mixing with the substrate components has been identified.

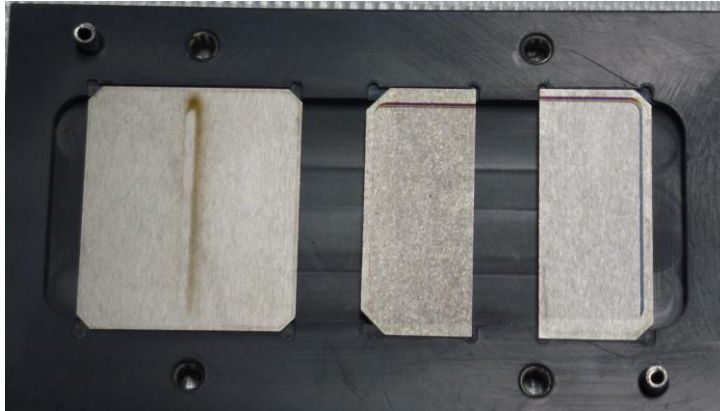
Overview – JET’s main chamber PFCs: Recessed Inner wall – Sticking monitors



OCT3



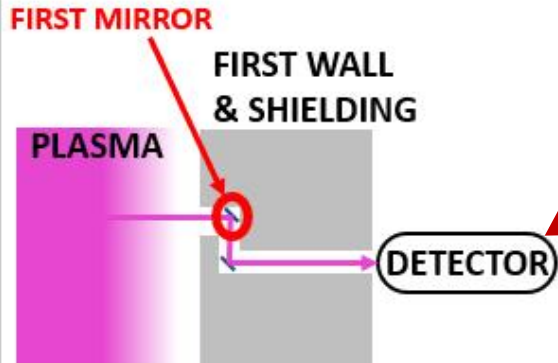
OCT5





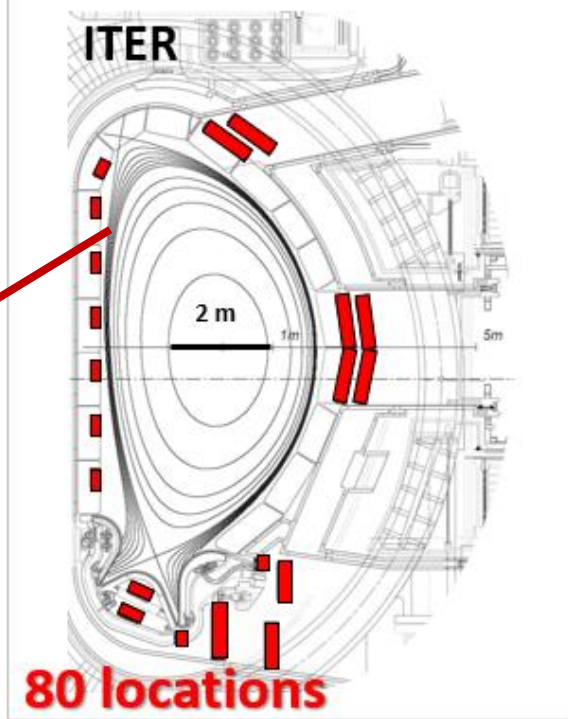
Overview – JET’s main chamber PFCs: Recessed Outer wall – Mirrors

CONCEPT

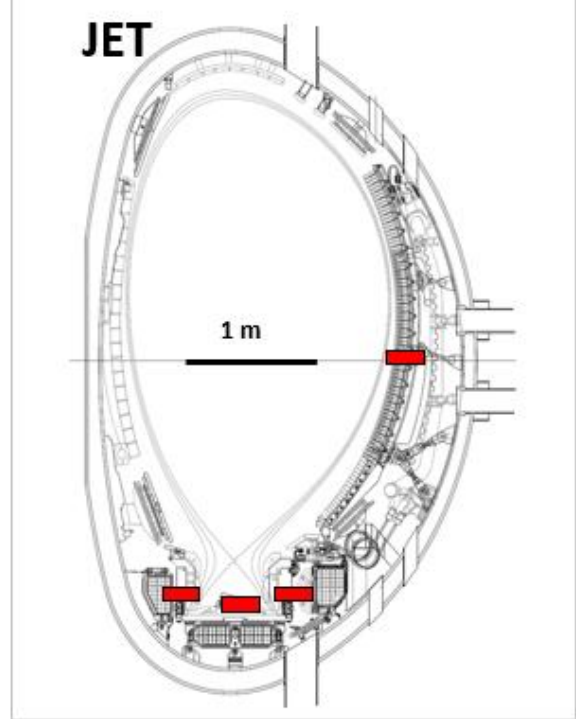


Protection against neutrons

PLAN



TEST

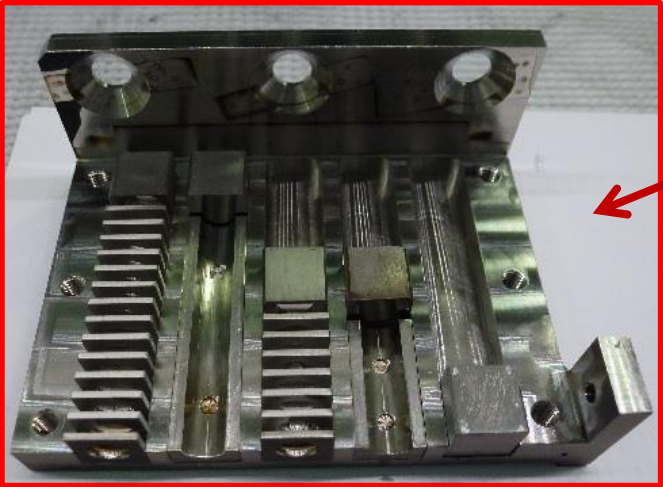




Overview – JET’s main chamber PFCs: Recessed Outer wall – Mirrors

CONCEPT

FIRST MIRROR



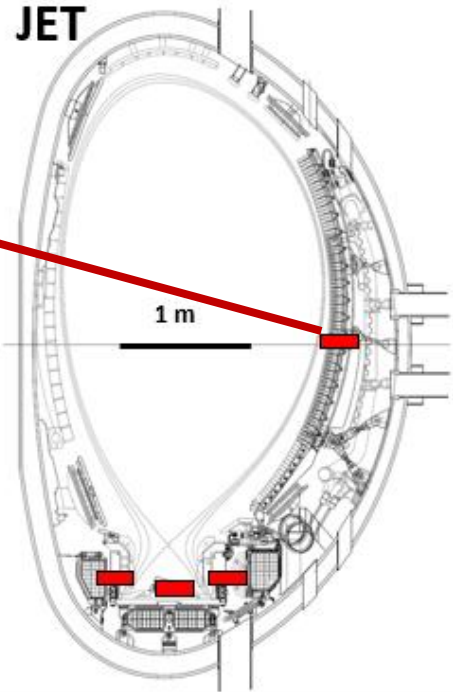
ITER

80 loc



TEST

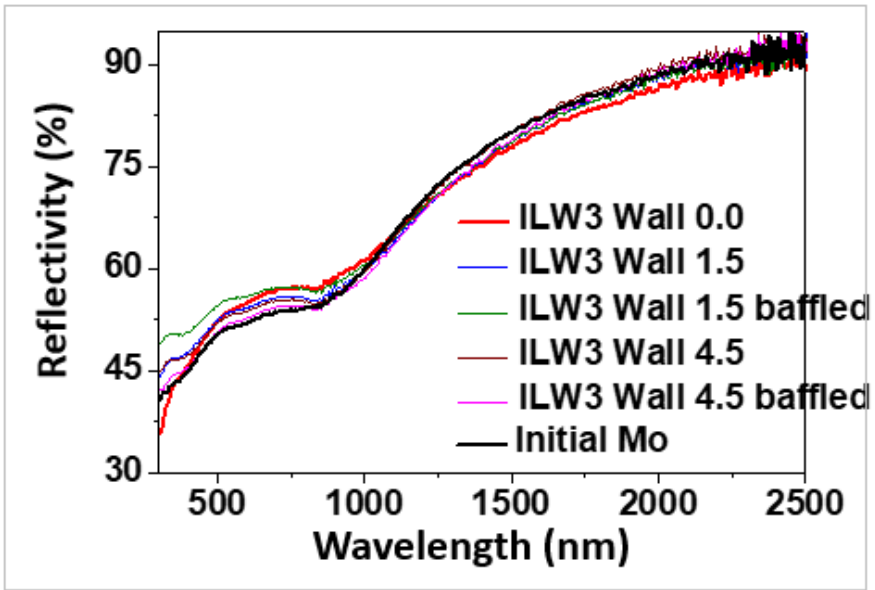
JET



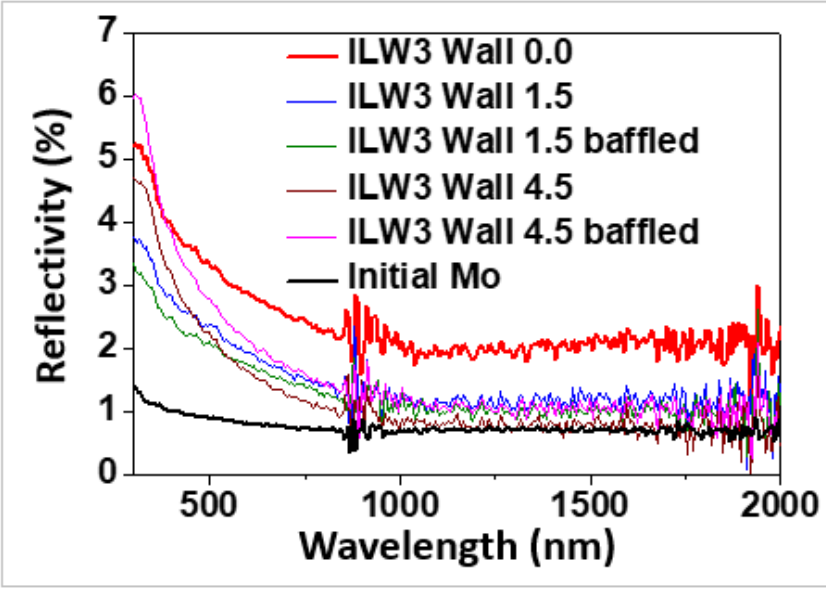


Overview – JET’s main chamber PFCs: Recessed Outer wall – Mirrors - Reflectivity

TOTAL reflectivity



DIFFUSE reflectivity

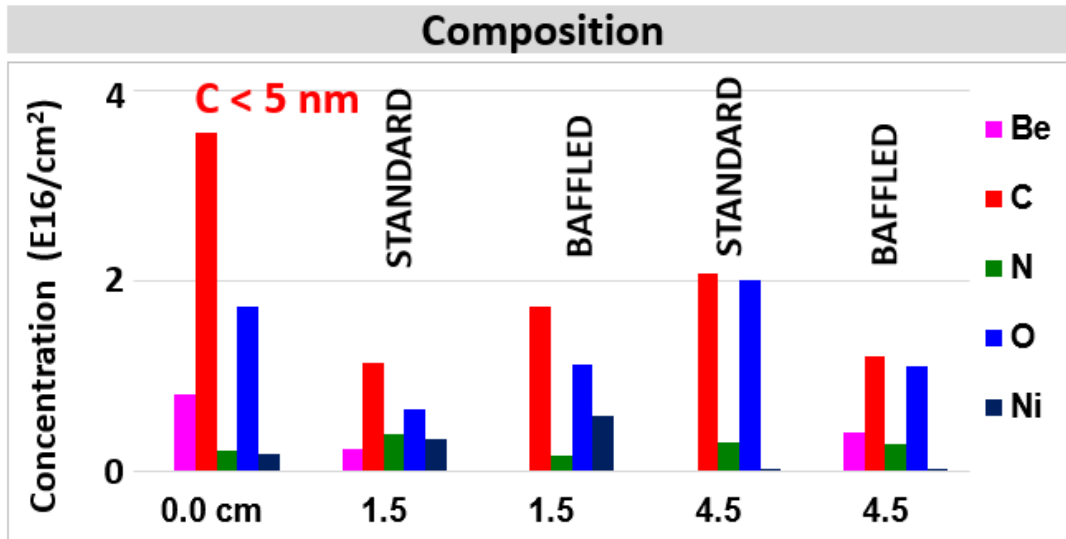


- Small changes of total reflectivity (*position independent*)
- Increase of diffuse reflectivity (*especially at the cassette mouth*)

[10] S. Moon, et al. Nuclear Materials and Energy 19 (2019) 59–6



Overview – JET’s main chamber PFCs: Recessed Outer wall – Mirrors - Surface



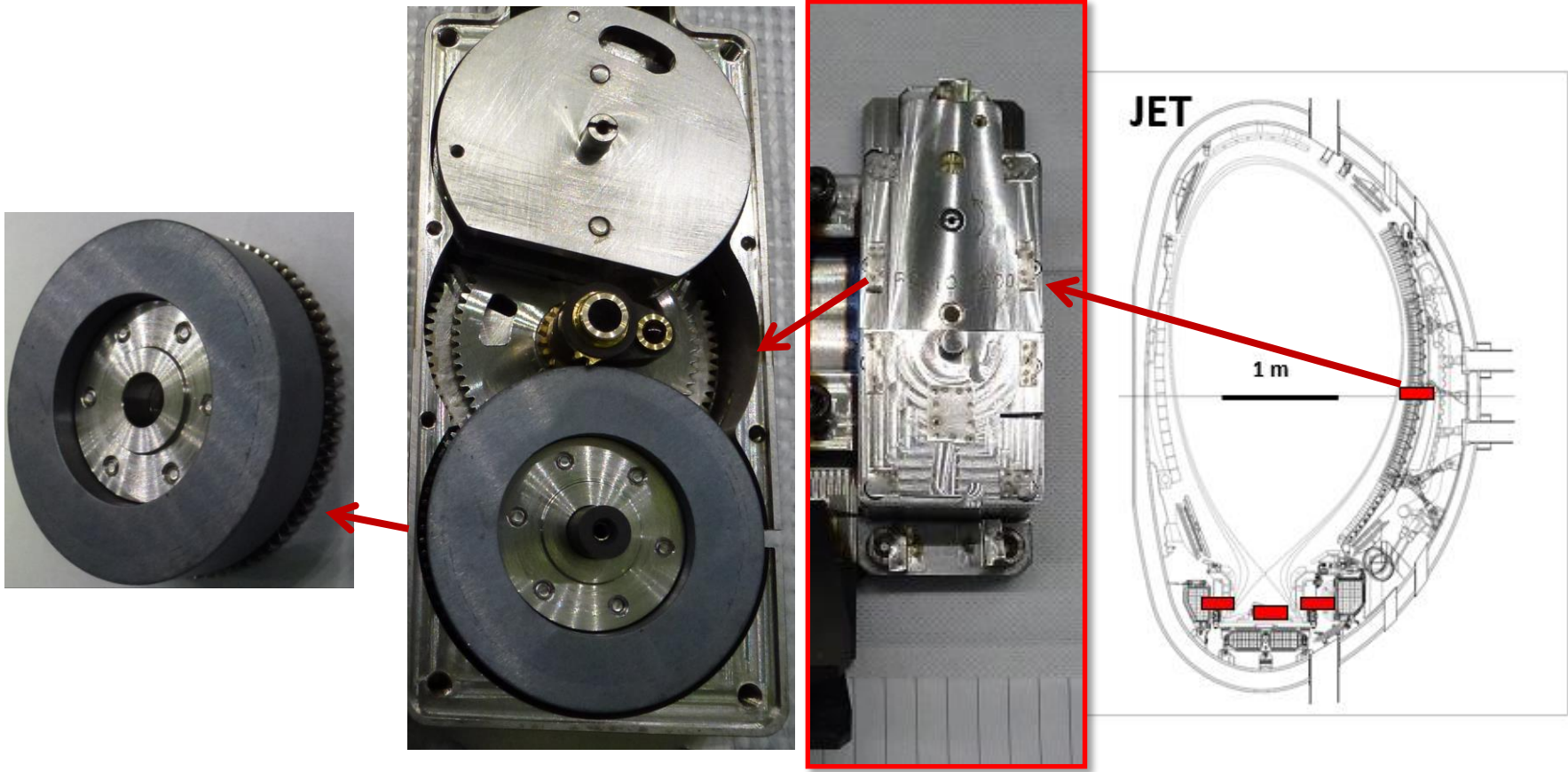
Roughness

Mirror	R _a (nm)
0.0 cm	8.48
1.5 cm	5.77
1.5 cm baffled	6.20
4.5 cm	4.87
4.5 cm baffled	4.72

- Composition: very small quantities of C, N, O, Be, Ni.
- Nitrogen is detected on all mirrors (and on all analysed PFC).
- No difference in deposition between standard and baffled channels.



Overview – JET’s main chamber PFCs: Recessed Outer wall – Rotating Collectors

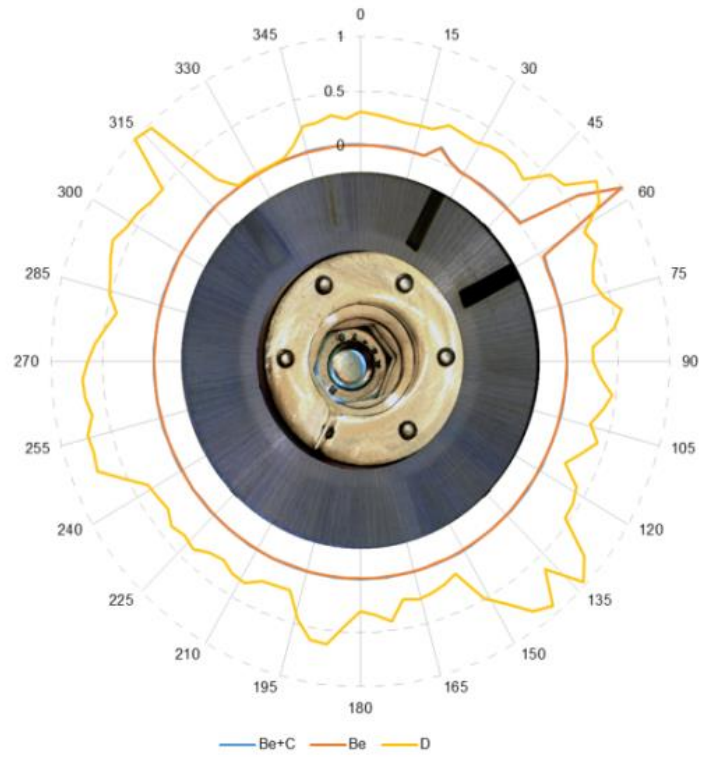


Overview – JET’s main chamber PFCs: Recessed Outer wall – Rotating Collectors

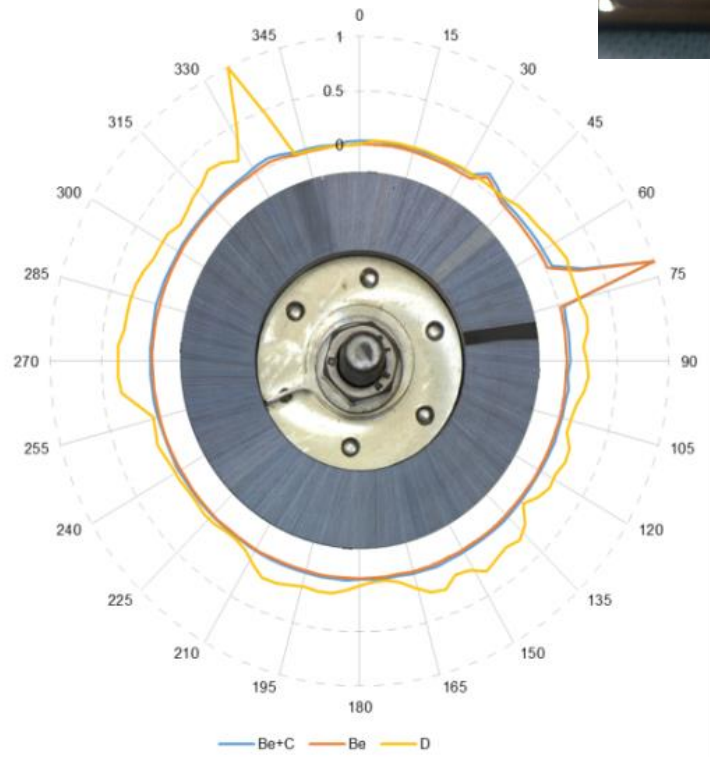


ILW-2

Rotating Collector 3E



Rotating Collector 4B

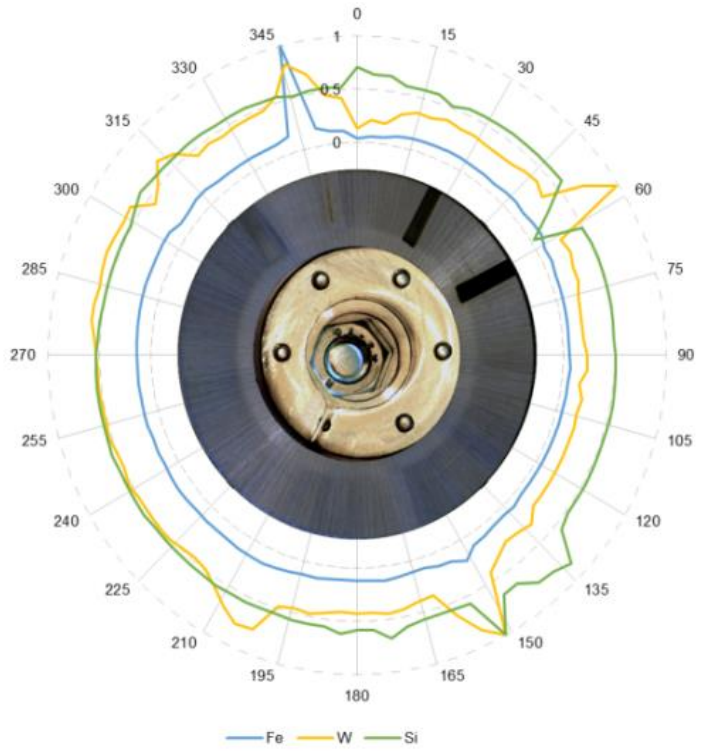


Overview – JET’s main chamber PFCs: Recessed Outer wall – Rotating Collectors

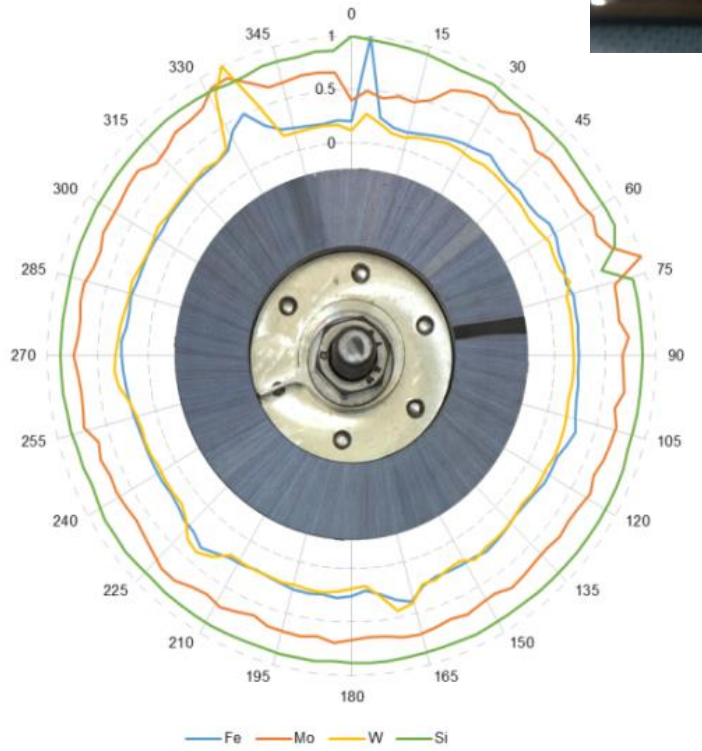
ILW-2



Rotating Collector 3E



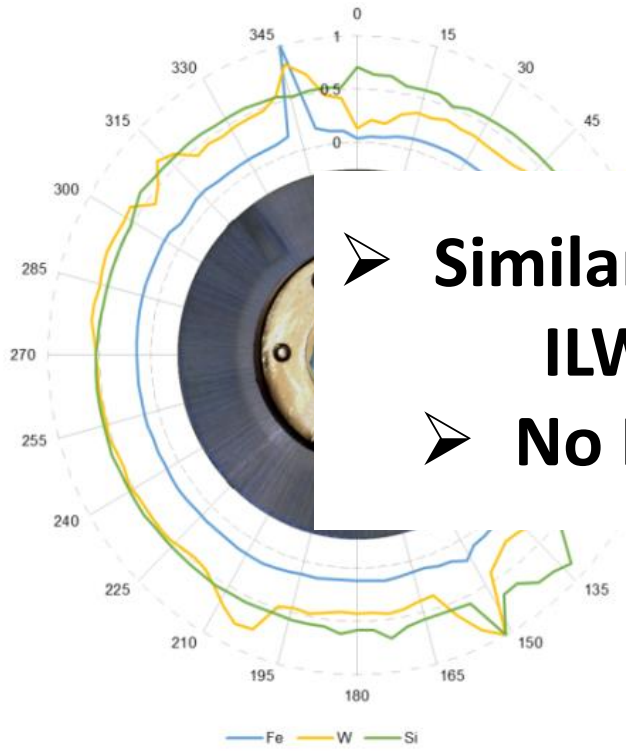
Rotating Collector 4B



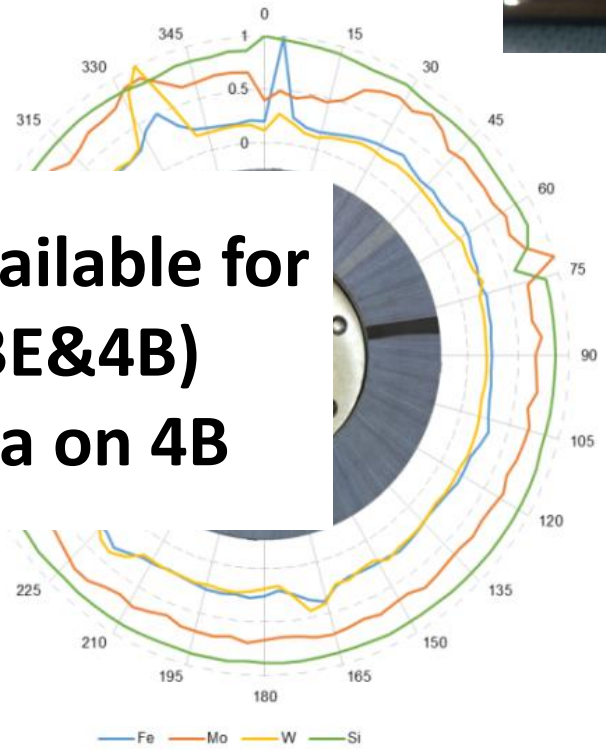
Overview – JET’s main chamber PFCs: Recessed Outer wall – Rotating Collectors



Rotating Collector 3E



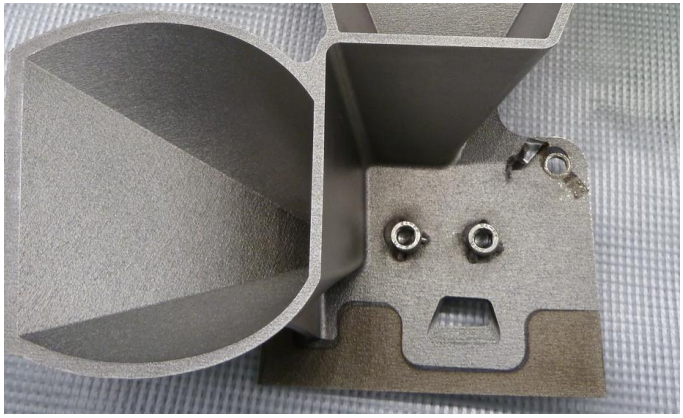
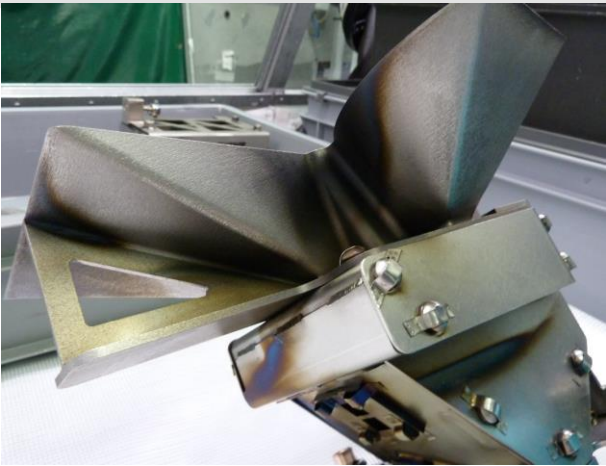
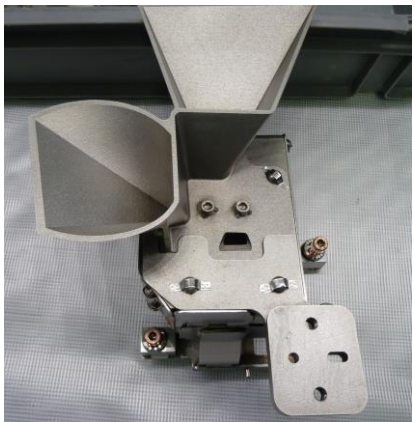
Rotating Collector 4B



➤ **Similar data available for ILW1 RCs (3E&4B)**
➤ **No ILW3 data on 4B**



Overview – JET’s main chamber PFCs: Recessed Outer wall – ITER mirror assembly





Overview – JET’s main chamber PFCs: Fuel retention and material migration in gaps of Be tiles

Fuel inventory and deposition in castellated structures in JET-ILW

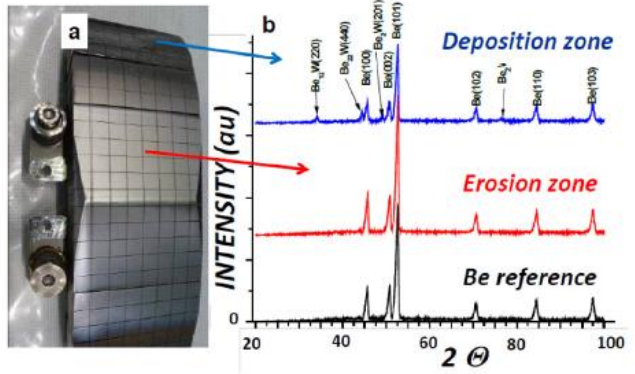


Figure 2. Castellated beryllium limiter tile from JET-ILW (a); x-ray diffractograms recorded for the initial limiter surface, erosion and deposition zones (b).

- Very shallow deuterium deposition is measured in the castellation: 0.5–1.5 mm deep into the groove
- Small quantities of D are found in the castellation both in the erosion and deposition zones. No difference is observed between the poloidal and toroidal gaps
- No dust accumulation was detected inside the castellation

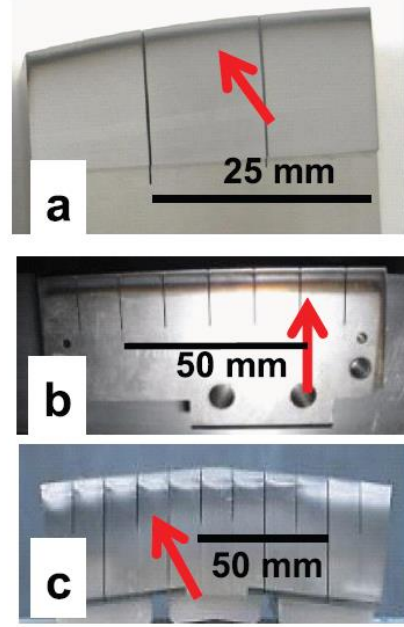


Figure 3. Deposition in the castellated groove (a); deposits of different width on the side surfaces of the tiles, in the gaps between the tiles (b) and (c).

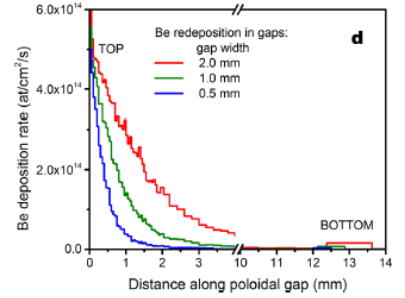
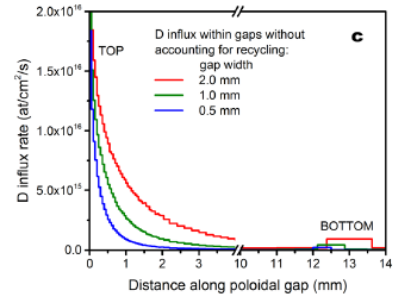
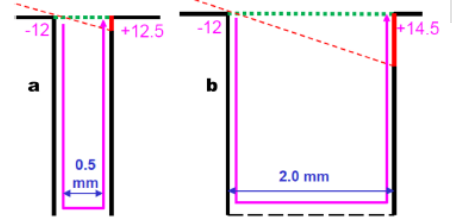
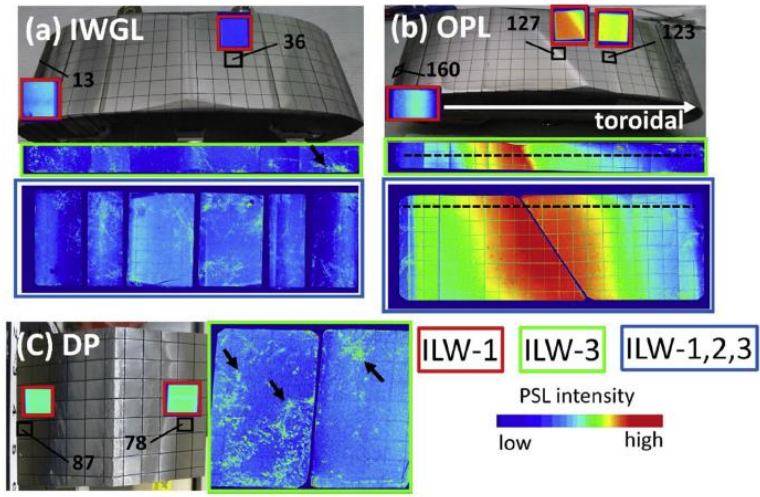


Figure 8. Modelling of deposition in the castellation: geometry of the gaps (a) and (b); simulated deposition profiles of deuterium (c) and beryllium (d) in the gaps of various widths.



Overview – JET’s main chamber PFCs: Fuel retention → difference between D and T in limiter tiles

Tritium distribution analysis of Be limiter tiles from JET-ITER like wall campaigns using imaging plate technique and β -ray induced X-ray spectrometry



- The observed T distributions on PFSs showed no systematic correlation with those of metallic impurities and D
- The highest T concentration was observed at the centre of OPL where the concentrations of D and metallic impurities showed the minimum values
- This difference in distributions indicated different deposition and retention mechanisms between T and D.

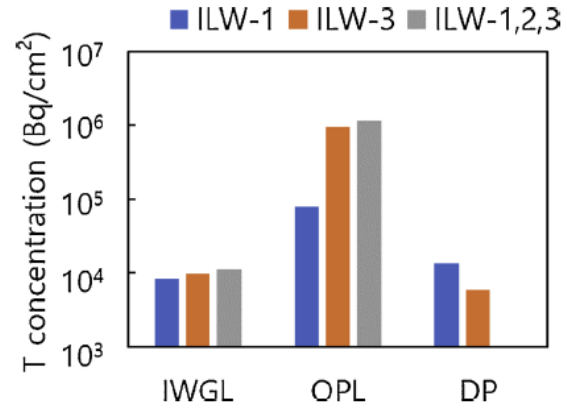
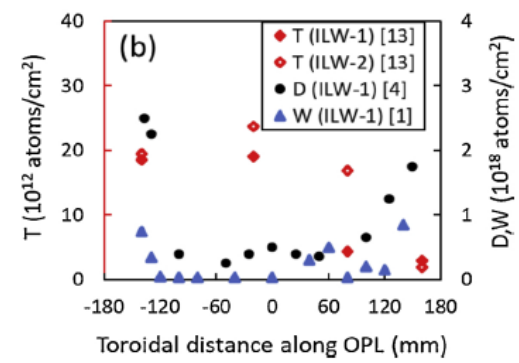
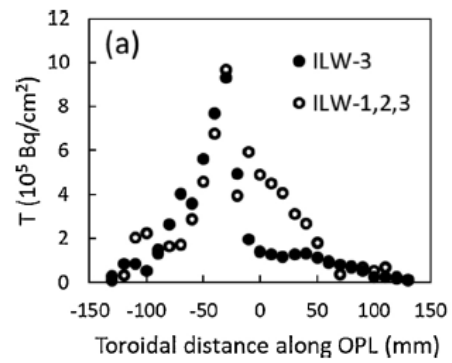


Fig. 6. Concentration of T evaluated from PSL intensity obtained by IP measurement. The regions with the highest PSL intensity were selected.



Conclusions

Be limiters

- IBA data for IL, OL and DP covering toroidal (throughout the “length” of entire tile) and poloidal distribution (bottom, mid and top part of JET vessel) for D and impurities (W, Ni, Cr, Fe) distribution for individual and combined campaigns available;
- Similar D distribution available via TDS analysis (not presented here)
- T assessment via IP measurements → different behaviour as compared with D distribution;
- D and impurities assessment in gaps available
- Melting due to VDEs and REs, splashes, material distribution around main chamber data available
- Microscopy, SEM, EDX, XRD, roughness available for pretty much all limiters (not presented here)

Recessed inner and outer walls

- Results from Inconel inserts available;
- Results from Be coated Inconel tile samples;
- RC results available.



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Thank you for your attention!

