

# "THEORY, SIMULATION, VERIFICATION AND VALIDATION"

# TSVV TASK 7: PLASMA-WALL INTERACTION IN DEMO

WP PWIE Meeting | 08.02.2023 D. Matveev on behalf of TSVV-7 team



### Aims of the project

Establish an integrated modelling suite capable to treat complex 3D wall geometry to predict steady-state PWI in DEMO

Provide safety-relevant information for DEMO reference scenarios concerning first-wall erosion, dust, and fuel inventory

Develop and apply modelling capabilities to treat PWI in DEMO-relevant transients regarding their impact on PFC integrity





### **Objectives**

Assessment of steady-state W erosion rates for first wall and divertor

Mapping of preferential W re/co-deposition locations

Assessment of dust mobilization from likely dust production sites (dust survival rates and dust accumulation maps)

Assessment of PFC response to transients: melting and splashing (melt-stability, likelihood of splashing, droplet-to-dust conversion rates)

Assessment of W erosion rates for locations affected by transients

Assessment of tritium in-vessel inventory

(co-deposition, bulk retention with He-induced and neutron damage)







#### PWI in DEMO

#### Team codes & contributions





### Team members

FZJ: D. Matveev, C. Baumann, J. Romazanov, S. Brezinsek KTH: S. Ratynskaia, L. Vignitchouk, P. Tolias, E. Thoren, P. Konstantinos IPP-CAS: D. Tskhakaya, M. Komm, A. Podolník USPN: J. Mougenot, Y. Charles CEA: R. Delaporte-Mathurin, E. Hodille, C. Grisolia, F. Montupet-leblond, IPP: U. Von Toussaint, K. Schmid Helsinki: F. Granberg, F. Kporha JSI: J. Kovačič, S. Costea





D. Matveev | WP PWIE Meeting | 08.02.2023

 $(6)$  :  $|S|$ C22 Sorbonne **PP** 

# TSVV-07 Timeline



**Report DEMO CDR** 

**Final report** 



#### Tasks 2022

#### Full 3D ERO2.0 simulations using existing PWI databases, sheath models and the plasma solution

Wall geometry, plasma equilibrium (baseline 2017), SOLPS-ITER plasma solution [F. Subba et al 2021 Nucl. Fusion 61 106013], neutral fluxes (not yet angular and energy resolved), magnetic shadowing and extrapolation to the wall implemented in ERO2.0





#### Tasks 2022

#### Full 3D ERO2.0 simulations using existing PWI databases, sheath models and the plasma solution

Wall geometry, plasma equilibrium (baseline 2017), SOLPS-ITER plasma solution [F. Subba et al 2021 Nucl. Fusion 61 106013], neutral fluxes (not yet angular and energy resolved), magnetic shadowing and extrapolation to the wall implemented in ERO2.0



(9)

#### Tasks 2022

#### Full 3D ERO2.0 simulations using existing PWI databases, sheath models and the plasma solution

First preliminary simulations of W erosion





Here: globally constant ion impurity concentrations based on integrated volumetric SOLPS data

- $D^+$   $\sim$  0.855 • He<sup>2+</sup>  $\sim$  0.134
- 
- He<sup>+</sup>  $\sim 0.008$
- $Ar^{13+}$  ~ 0.003

(mean charge state of Ar)



© C. Baumann



#### Ongoing work

#### Full 3D ERO2.0 simulations using existing PWI databases, sheath models and the plasma solution

Accounting for charge state resolved impurity concentrations and fluxes



! Recent ERO2.0 update has introduced fully charge state resolved impurity concentrations and respective PWI, so that new erosion results are coming soon

! SOLPS-ITER solution with extended grid will be very important – is pending (DCT)





#### Tasks 2022

#### Dedicated PIC studies with BIT-1 for DEMO-relevant high-density divertor sheath as relevant input for erosion, dust transport and transient melting simulations

Simulations for DEMO relevant divertor sheath with densities up to  $5x10^{21}$  m<sup>-3</sup> indicate that neutrals dominate heat loads

The angular distributions of impacting ions are strongly affected in the collisional sheath and acquire the shape similar to the distribution of neutrals for the highest density cases



Characteristics of the high density divertor sheath, such as plasma parameters and impact energies and angles of ions and neutrals are available for implementation in ERO2.0 for simulations of divertor erosion





#### Tasks 2022

#### MD simulations utilizing available W-H interatomic potentials to simulate supersaturated W surfaces under ion irradiation

- Yet static only simulations due to forced usage of slow Tersoff-type interatomic potential
- Studied pure and supersaturated W irradiated by D and  $D_2$  ions  $\rightarrow$  no difference due to molecules
- 5%atD, 10%atD, 20%atD
- 0º, 30º, 60º angles of incidence
- Supersaturation decreases the sputtering yield that seems to be not due to dilution, neither enhanced reflection, work in progress

### Sputtering yield at different incoming angles for different D levels in the material 0.007 © F. Granberg 0,006 0,005 Sputtering yield<br>0,003<br>0,003 0,002 0,001  $\Omega$

● Pristine ● 5 % ● 10 % ● 20 %

30

Incoming angle

40

10

 $\Omega$ 

20



50

60



#### Tasks 2022

#### Dedicated dust transport simulations with MIGRAINe (aiming at net deposition locations provided by ERO2.0 simulations)

Based on 2021 MIGRAINe simulations using ITER geometries and low-power plasma profiles, estimates of long-term W dust inventory evolution in 12 remobilization scenarios were refined [Vignitchouk 2022]

Simple Markov chain fit models improved to sub-percent errors

Dust distributions shift to larger sizes as time evolves (small dust does not survive in plasma due likely vaporization)

Sampled DEMO plasma profiles are similar to ITER high recycling conditions considered for the dust survival studies

Work in progress: MIGRAINe modifications to allow multiple impurity species and ion charge states are in progress

Simulations for ERO2.0 deposition location maps expected in 2023

Sampling of plasma conditions from DEMO plasma profiles (Te)







#### Tasks 2022

#### PIC simulations with SPICE to identify whether the escaping current scales with electron thermal velocity or on ion sound speed

Simulations of thermionic emission (TE) with secondary electron emission (SEE) and electron backscattering (EBS) confirm the validity of the earlier developed semi-empirical scaling model [Komm 2020, Tolias 2023]

In the case of high density emissive sheath and normal magnetic field inclination that is relevant for exposed leading edges, the transition to the space-charge limited regime will not take place before W melting, thus implying that the monotonic potential profile regime is most relevant for melt motion applications

#### Implementation of representative values of surface heat fluxes and halo current densities scaling [MA/m<sup>2</sup>] for transient melting simulations with **MEMOS-U MEMENTO**

The MEMOS-U model was re-implemented in a new code using the open-source AMReX adaptive meshing framework

For current DEMO design, the thermal and the current quench of VDEs are considered relevant for melting (WPDES, late 2022) – assessment of melt layer stability in these conditions is pending





#### Tasks 2022

#### Retention and permeation studies for DEMO main chamber and divertor with TESSIM-X, MHIMS, FESTIM, RAVETIME

Highlights: implementation of Soret effect, introduction of source terms for n-induced defects, work on mobile dislocations, H-He interaction, simulations of the role of interface conditions and 3D effects for ITER-like W monoblocks



#### Tasks 2022

#### Implementation of the gyro-motion module in SDTrimSP-3D

Gyro-module implemented, optimization and implementation of crystalline lattice capabilities are on the way / validation

Future work: cross-check with ERO2.0, evaluation of sample exposures in AUG, cross-validation with MARLOWE and MD

#### Implementation of non-uniform injection schemes in SPICE

For the cases of non-uniform plasma density, temperature

#### Implementation of non-steady-state sources in BIT1 (blobs)



Steady-state vs Blob-filament  $Q_{total}^{I}/Q_{total}^{e+I}$  $\cdots$  0.0  $2.5$  $2.5$ (total power flux) 5.0  $5.0$  $7.5$  $7<sup>5</sup>$  $06$ © J. Kovacic  $\frac{1}{\times}$  10.0  $E_{10.0}$ 12.5 15.0 15.0 17.5  $17.5$ D. Matveev | WP PWIE Meeting | 08.02.2023 (17) $20.0$  $time[µs]$ 

time [us]



### Tasks 2023 (1/2)

- 1. Implement the MD erosion data under D/T supersaturation into ERO2.0.
- 2. Implement the PIC-inferred sheath characteristics into the ERO2.0 code.
- 3. Perform full 3D ERO2.0 simulations using updated databases and sheath models. New plasma?
- 4. Provide erosion-deposition maps and wall lifetime for the main chamber and divertor of DEMO.
- 5. Perform MEMENTO simulations of PFC response under VDEs. Assess macroscopic surface modifications in form of melting, melt re-solidification and possibly splashing. In case of splashing, analyze the size and velocity distributions of droplets.
- 6. Perform dedicated dust transport simulations with MIGRAINe for net deposition locations provided by ERO2.0 simulations.



### Tasks 2023 (2/2)

- 7. Perform SDTrimSP-3D simulations for rough surfaces of specific morphology and in particular that of re-solidified melt resulting from MEMENTO to obtain effective erosion yields.
- 8. Perform PIC simulations with SPICE codes to obtain heat loads and ion penetration into gaps of divertor and limiter monoblocks under DEMO sheath conditions.
- 9. Perform TESSIM-X and FESTIM simulations of fuel accumulation in DEMO PFC with relevant material structure and fuel permeation to coolant, accounting for neutral damage. Cross-validate the results of both codes.
- 10. Develop with help of ML approach an interatomic potential for W-O.
- 11. Prepare an extensive integrated report for the DEMO conceptual design review regarding the DEMO wall lifetime in steady-state and transients and regarding fuel uptake, retention and permeation.



### TSVV-07 Timeline



**Report DEMO CDR** 

**Final report** 

