



# Additive manufacturing as tool to manufacture and maintain plasma facing components

D. Dorow-Gerspach, M. Gago, J. Tweer, M. Wirtz



This work has been carried out within the framework of the EUROfusion Consortium, funded by the European Union via the Euratom Research and Training Programme (Grant Agreement No 101052200 — EUROfusion). Views and opinions expressed are however those of the author(s) only and do not necessarily reflect those of the European Union or the European Commission. Neither the European Union nor the European Commission can be held responsible for them.

- Publications / Fundamentals
- O.1. AM-W: AM of W with density >99% and minimal crack density
- O.2. Joints: Realizing advanced joints (FGM, W/W<sub>w</sub>, W/steel, W/copper)
- O.3. W-wire as armour: Realization and testing of W-wire as armour
- O.4. Regeneration: Development of techniques for surface regeneration
- O.5. Advanced heat sink geometries: Development, production and testing
- O.6. Demonstrators of used technologies: Construction of prototype mock-ups

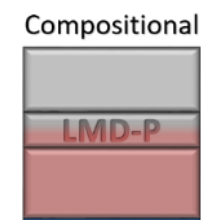
## Publications

- V. Ganesh – ID: 33546  
Processing and properties of sintered W/steel-composites for the first wall of future fusion reactor  
<https://doi.org/10.3390/jne4010014>
- V. Ganesh – ID: 33870  
High heat flux testing of graded W-steel joining concepts for the first wall  
<https://doi.org/10.3390/en16093664>
- V. Ganesh – ID: 33870  
Determination of mechanical properties of tungsten/steel composites using image based microstructure modelling  
<https://doi.org/10.3390/en16093664>
- D. Dorow-Gerspach – ID: 33811  
Benchmarking by high heat flux testing of W-steel joining technologies  
<https://doi.org/10.1016/j.nme.2023.101508>
- J. Tweer – ID: 36194  
First experiments to regenerate the surface of plasma facing components by wire based laser metal deposition  
<https://doi.org/10.1016/j.nme.2023.101508>

## Fundamentals

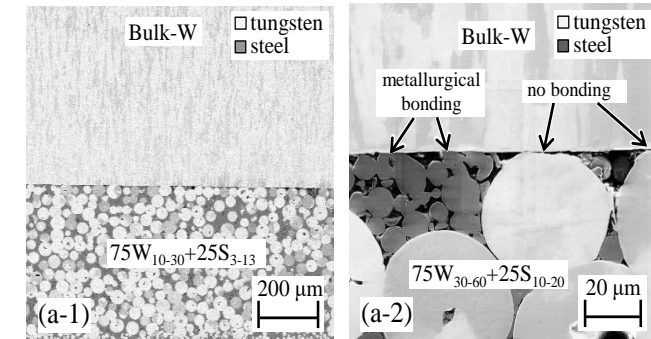
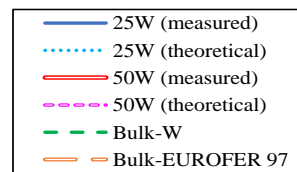
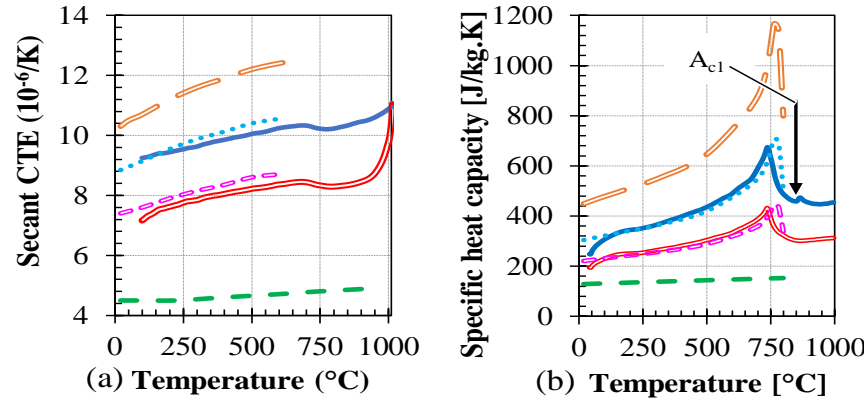
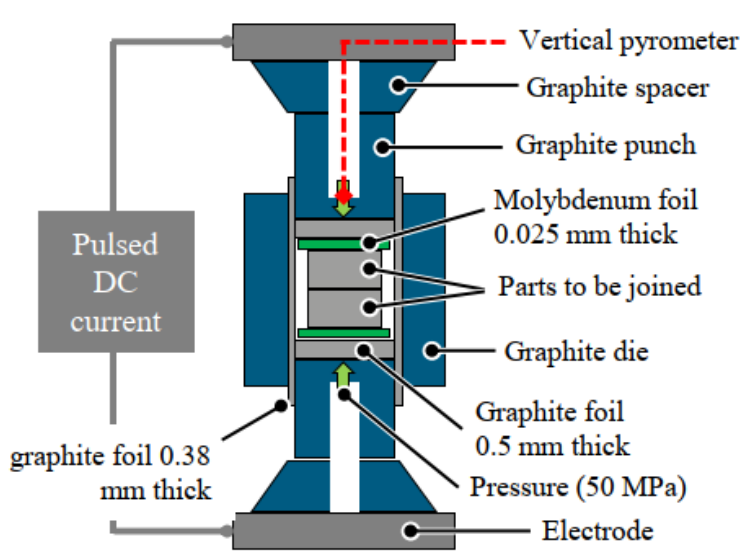
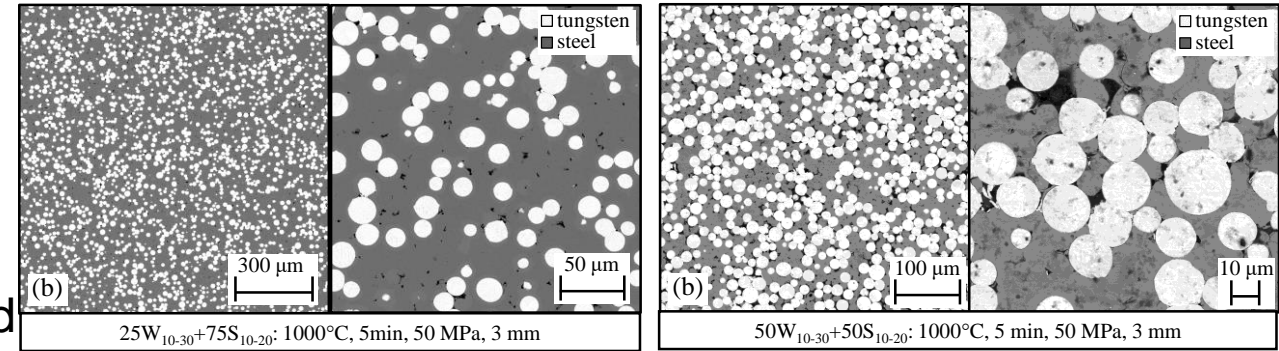
- Financial limitations causing severe limitations on experiments and slowing down progress in nearly all objectives
  - Except: W-Regeneration by LMD-w
- Colleagues from KIT faced also major delays thus they couldn't support us with SEBM-W samples
  - The geometrical joint concept couldn't be realized let alone tested yet
- Prototype mockups with 25x25 mm<sup>2</sup> instead of standard 12x12mm<sup>2</sup> tile sizes will be realized and tested this year

# O.2. Joints – Compositional FGMs

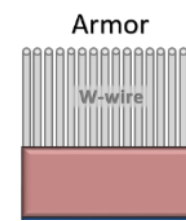


- Various different methods were used
  - APS, EDS, SPS producing single & full stack
  - First benchmark test indicated that thin metal film together with SPS-FGM should outperform ref.
  - New samples were made with 10  $\mu\text{m}$  V or Ti film
  - Benchmarktest including W<sub>f</sub>W on steel are planned

## Spark Plasma Sintering



# O.3. W-wire as armor



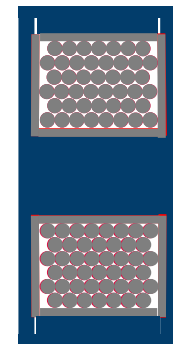
## ➤ Larger spool design

- Enough to cover 460 cm<sup>2</sup>, 3 mm thick
- Cylindrical shape did not improve stacking quality or thickness before losing wires
- More effort to form plan parallel slices
- ➔ **Cylindrical design dismissed**



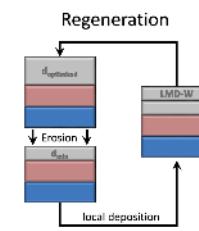
## ➤ New Design

- Rectangular with half circle at the end
  - Larger portion of wires easily useable
- Inlet of 125 μm W-foil as “wall” for better mechanical stability at the sides
- **Spool is built, foil and wire are ordered**



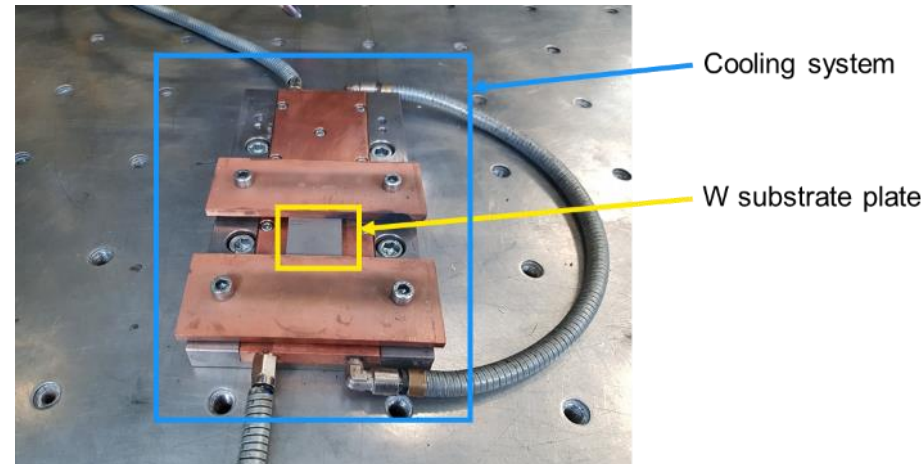
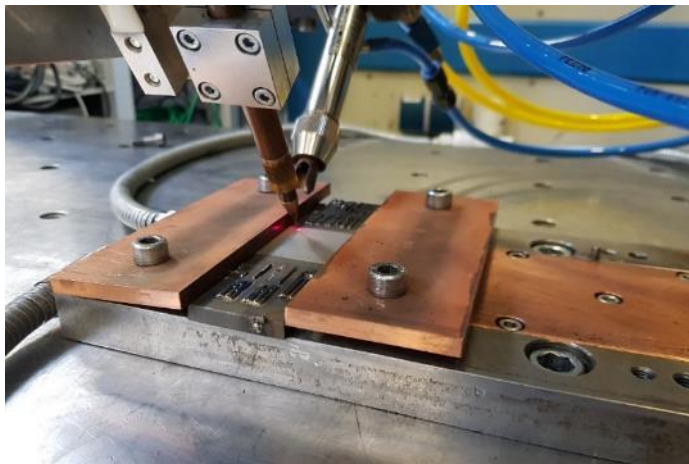
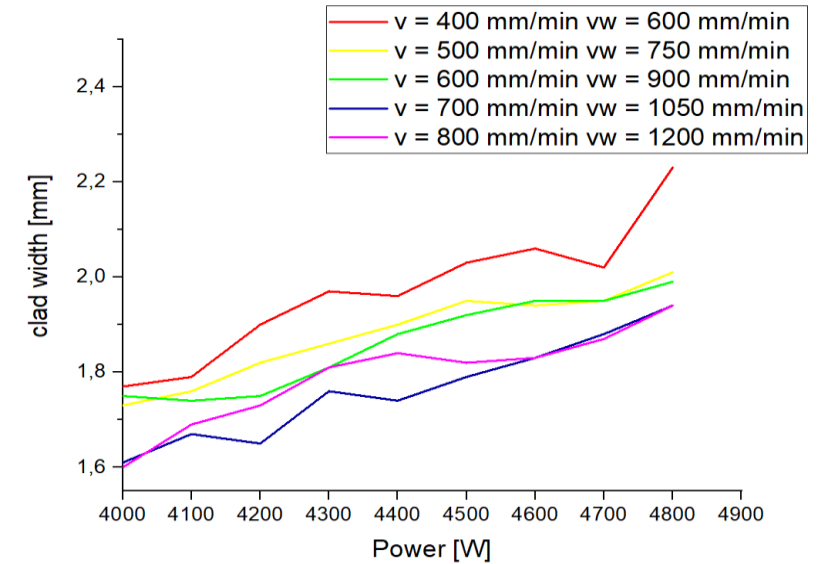


# O.4. Regeneration



## LMD-W at IPT Aachen

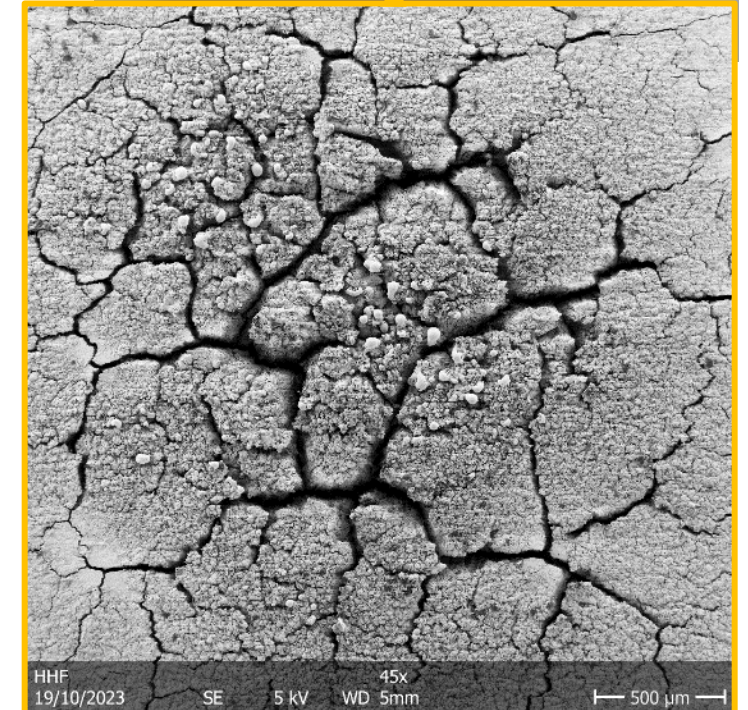
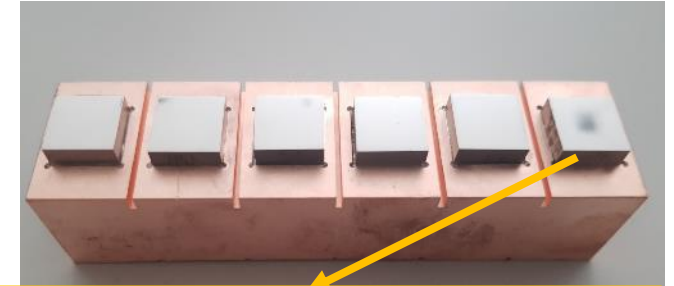
- Ar flow, 4 kW IR laser, W on steel, W, W<sub>w</sub>
- Porosity, Cracks have to be reduced
- Cooling holder to improve substrate temperature control during the deposition
  - Parametric studies are performed -> work ongoing
  - Width nicely tailorable by power and velocity



# O.4. Regeneration

## • Predamaging in JUDITH 2

- 12 x 12 mm<sup>2</sup> W-tiles on Cu-cooling brazed
- Tansient heat load: 10<sup>5</sup> Thermoshocks, 0.5 ms with  $L_{abs} = 0,55 \text{ GW/m}^2$  ( $F_{HF} = 12 \text{ MWs}^{0,5}/\text{m}^2$ ) at base temperature of  $T_{base} = 700^\circ\text{C}$

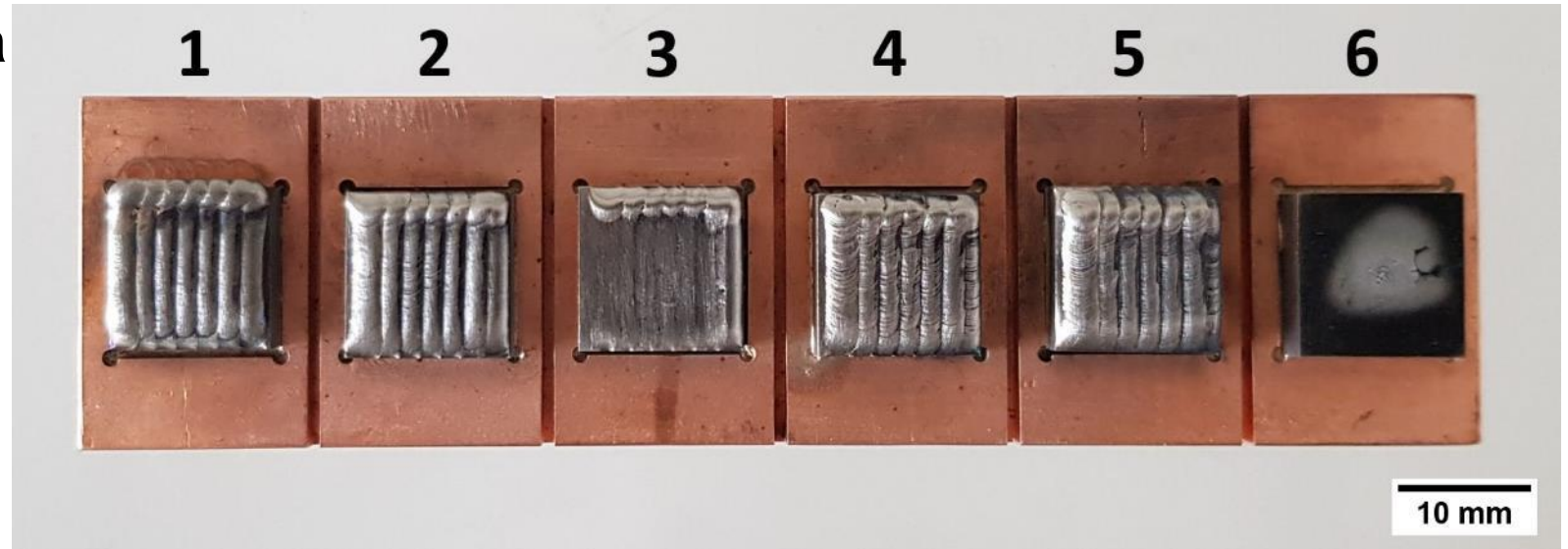




# O.4. Regeneration

## Deposition on damaged area

- 1: Layer with standard condition
- 2: Lower deposition velocity
- 3: Remelting damaged area
- 4: Remelting + standard Layer
- 5: Remelting + lower velocity layer
- 6: Reference



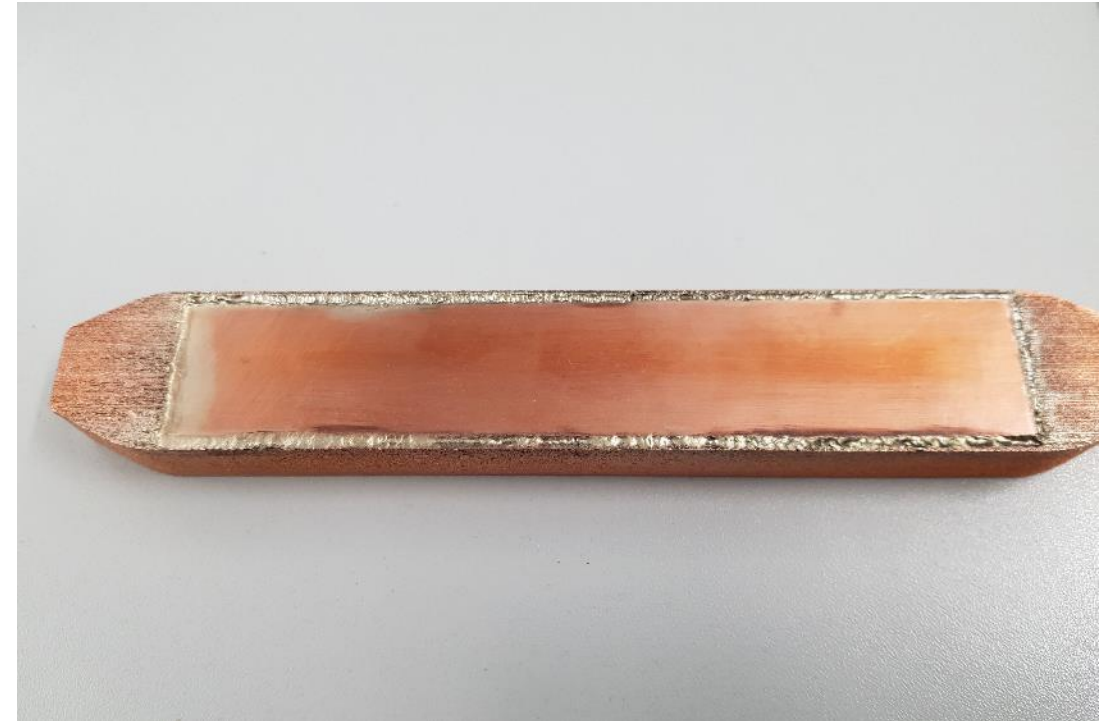
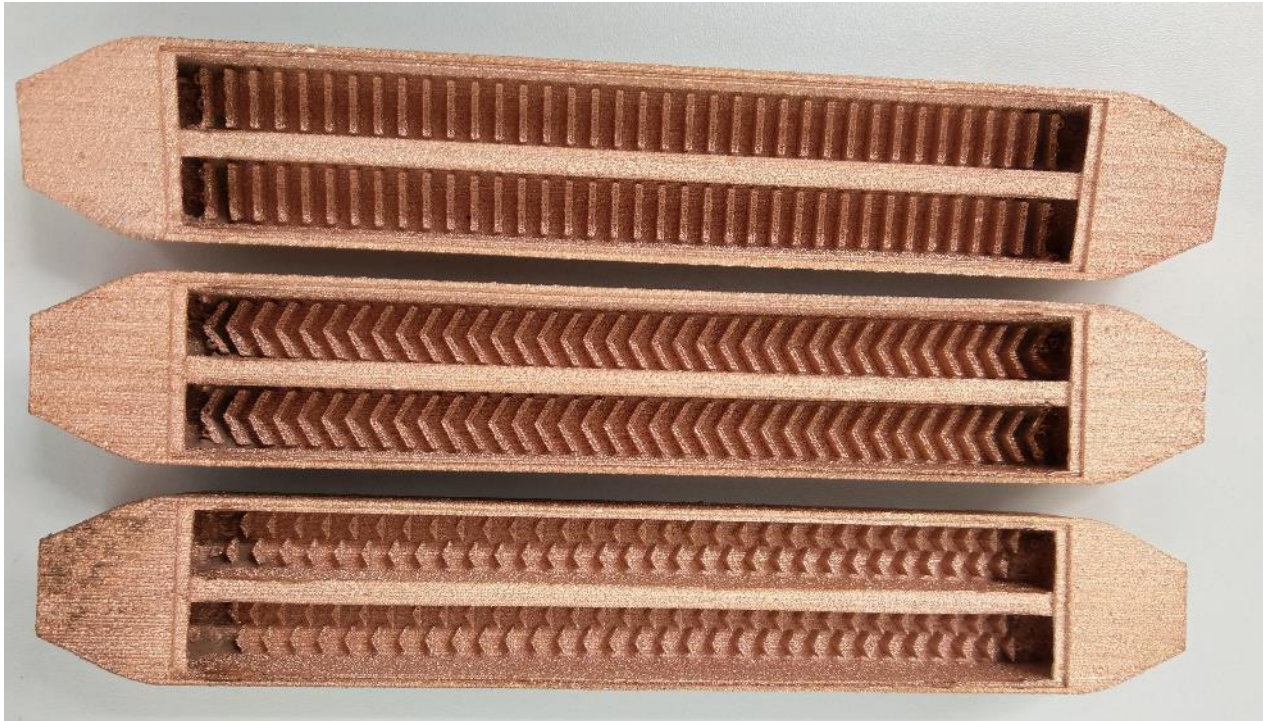
## Metallographic analysis ongoing

- Cross sections at start, center and end of each weld bed
- Microstructural investigation including hardness measurements



# O.5. Advanced heat sink geometries

- Further geometries with Cu was printed at IFAM
  - Hydrodynamic and high heat flux tests to assess potential and impact of designs are planned
  - Simple thermal comparative FEM simulations to assess HTC and disentangle area from flow dynamics
  - Backplate **soldering failed**, **two welding** tries **failed** as well, a last try with higher energy is ongoing



# O.6. Prototype mock-up: LMD-w on W

Mitglied der Helmholtz-Gemeinschaft



**W-tiles: „monoblock pair“, 8 (reference) or 7 mm thick (for deposition)**



**Cu-cooling structure fitting JUDITH 2 clamping**



**PFC for HHF testing of LMD-w deposits**

# O.6. Testing conditions for a LMD-w PFC

- Test conditions to mimic divertor condition
  - Aiming for the same surface temperatures at
  - Elevated cooling water temperature and 8 mm thick W-tiles for comparable temperature profile
  - **Simulate the center of a MB with flat tile design**
- 2 Components are planned
  - One layer with and w/o remelting
  - Two layers with 0 and 90° between depositions
  - 10 MW/m<sup>2</sup>, 15 MW/m<sup>2</sup> and 20 MW/m<sup>2</sup>
    - Each at least 200 cycles
  - Transient loadings on deposited material

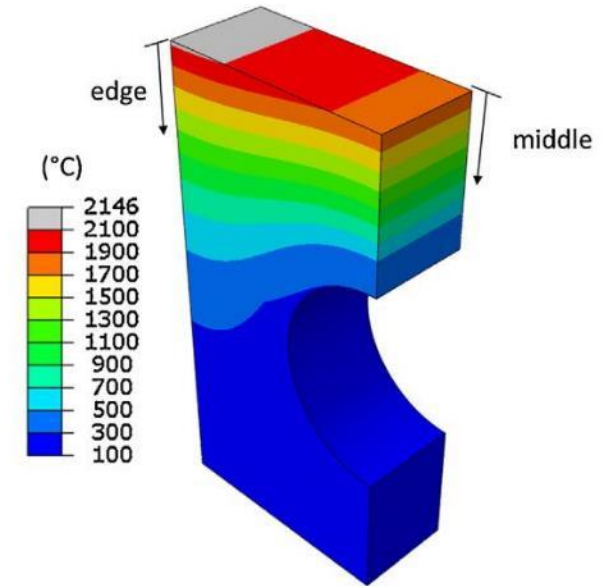


Fig. 4. Temperature field in the divertor target under stationary HHF load of 20 MW/m<sup>2</sup>.

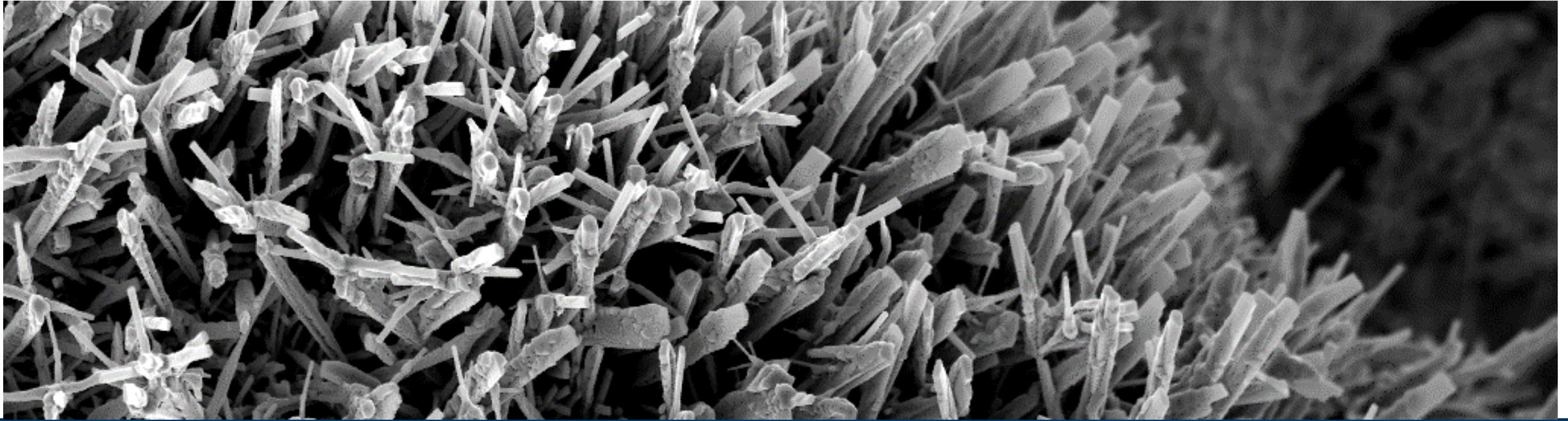
Table 3

Calculated temperature range on the armor surface during steady state HHF loading.

Heat flux loads	Temperature ranges
10 MW/m <sup>2</sup>	950–1058 °C
15 MW/m <sup>2</sup>	1411–1612 °C
20 MW/m <sup>2</sup>	1864–2146 °C

M. Li and J.-H. You; Nuclear Materials and Energy Nuclear Materials and Energy, vol. 14, pp. 1-7, 2018, doi: 10.1016/j.nme.2017.12.001





# Thank you for your attention

Outtakes