

Pre-conceptual design of LMD tiles for COMPASS-U:

**Activities in 2023
&
Plans for 2024**

R. Dejarnac on behalf of IPP.CR

+

[UKAEA, UK] & [IST, Portugal] & [CIEMAT, Spain]

8.2.2024



MINISTRY OF EDUCATION,
YOUTH AND SPORTS

- Status of the COMPASS-U project
- Activities in 2023
 - Design
 - Modeling
- LMD plans for 2024
 - Design / Experiments
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COMPASS-U is a fully new device under construction in IPP Prague

Main parameters

- Toroidal magnetic field
- Plasma current
- Major radius
- Minor radius
- Metallic (inertial) first wall + Divertor manipulator
- Vacuum vessel operation temperature up to 500°C

$$B_t = 5 \text{ T}$$

$$I_p = 2 \text{ MA}$$

$$R_g = 0.9 \text{ m}$$

$$a = 0.27 \text{ m}$$

Plasma heating power

- Initial phase
- Later phase
- 3 s flattop discharges

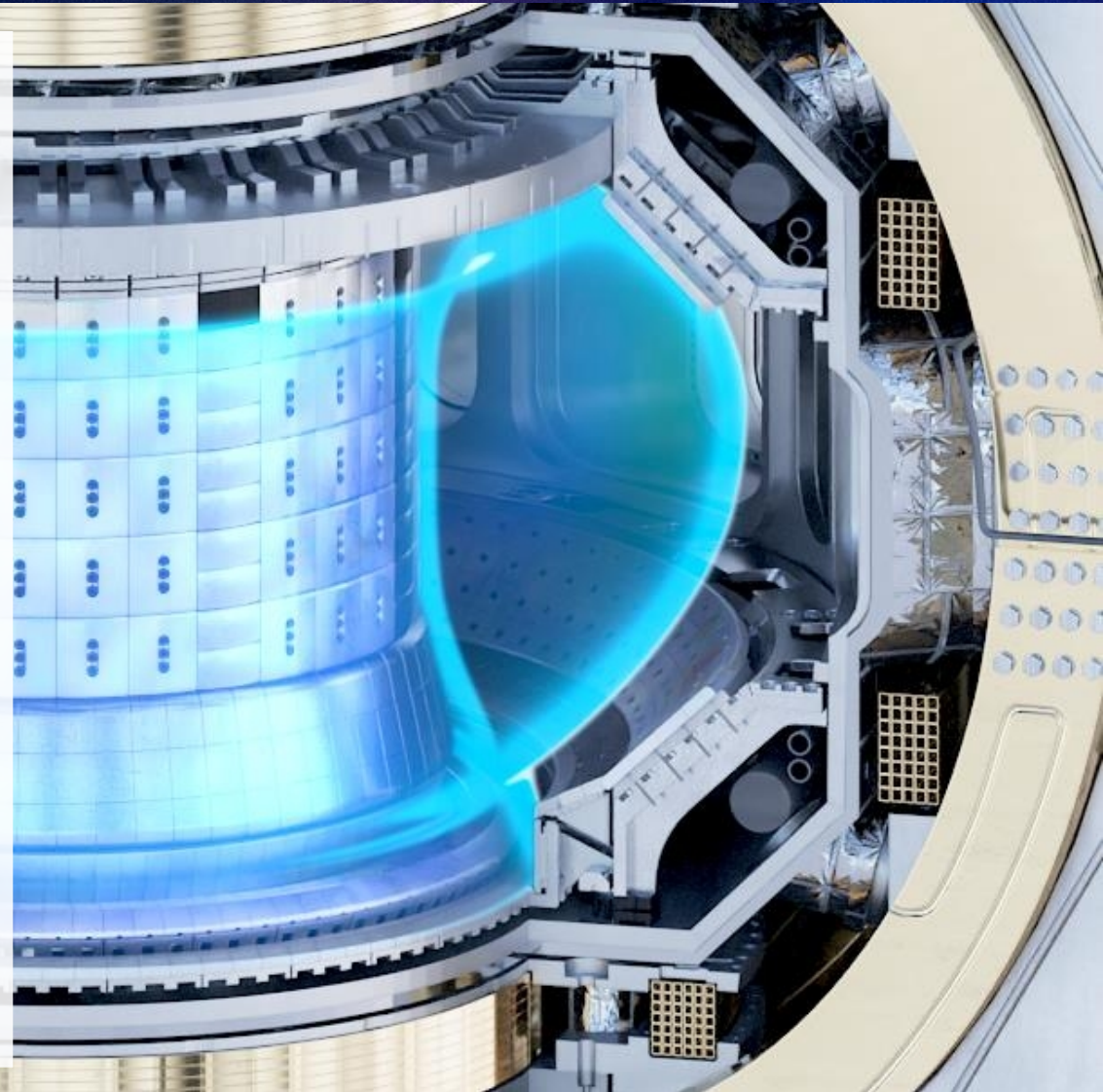
$$P_{\text{NBI}} \geq 3 \text{ MW}, P_{\text{ECRH}} = 1 \text{ MW}$$

$$P_{\text{NBI}} = 6 \text{ MW}, P_{\text{ECRH}} = 8 \text{ MW}$$

Scientific objectives

- Power exhaust + Advanced confinement + LM (full-ring divertor)

First plasma expected in 2026





Torus hall



Installation of IGBT power supplies for PF coils



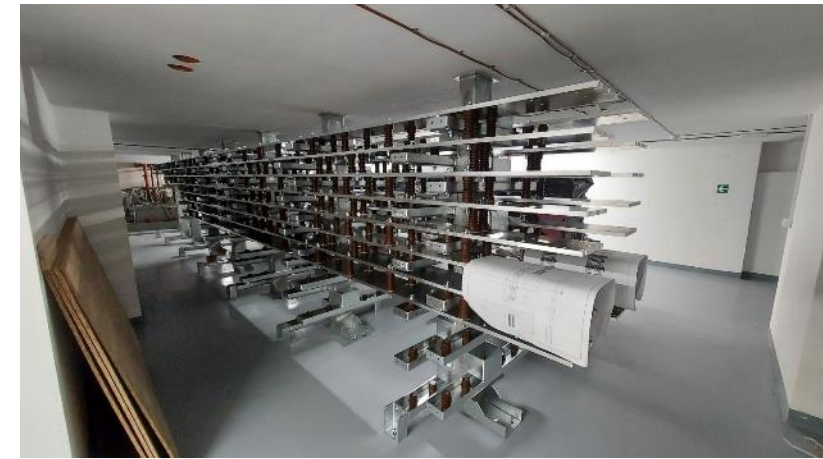
TF coils thyristor converters



Assembly hall



1st fly-wheel during the FAT tests



Installation of the PF coils bus-bars



Three-floor new hall for power supplies, diagnostics and gyrotrons



Basement for two fly-wheel generators

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Conceptual Design in collaboration with UKAEA



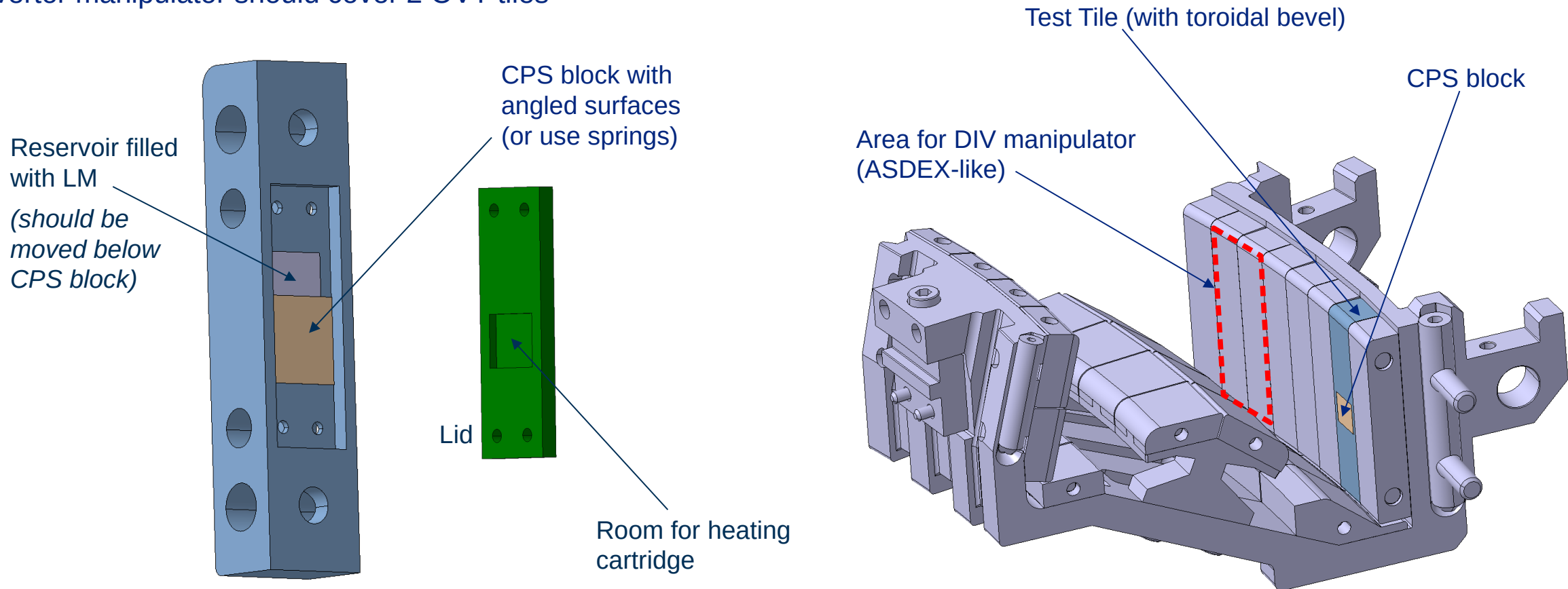
- Team led by **D. Horsley**
- **CAD models of COMPASS-U OVT tile + support structure** were shared w/ mechanical eng. (M. Bastar)
- **Documents based on DEMO design templates** were produced:
 - Stakeholder Requirements Document (SHRD) to identify and capture the stakeholder requirements for the COMPASS-U LMD tile design and future experiments – **C-U will be test bed for LM and open for EF experts.**
 - Outcomes Requirements Document (ORD) to define the expected research outcomes of the future COMPASS-U experiments using the LMD tile as these outcomes should influence its design.
 - Design requirements against which any future design will be compared.

Workshop organized (18.01.2024)

- Discuss the design strategy and requirements prioritization (*technology demonstration + extrapolation to DEMO*)
 - Main goal? Is it engineering or physics?
 - **Engineering:** Mechanical robustness, best CPS techno (mesh, felt, 3D-printed), hard stops, which LM (?), replenishment, cold trap, etc...
 - **Physics:** Erosion, redeposition, vapor shielding efficiency, core pollution, validation of modeling (how?), etc...
- Comment on the present conceptual design

Design Concept *(brief overview – see D. Horsley's talk)*

- A pre-conceptual design of a “CPS block” and its implementation in one COMPASS-U OVT tile was created
- Divertor manipulator should cover 2 OVT tiles



2023 modeling activities

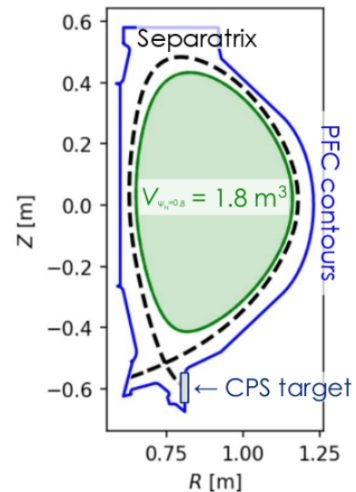
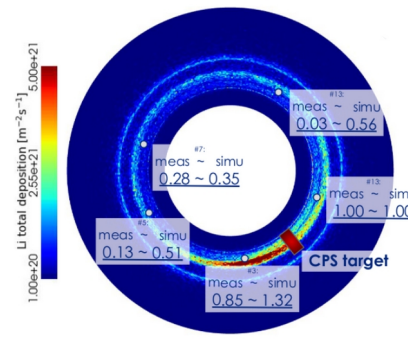
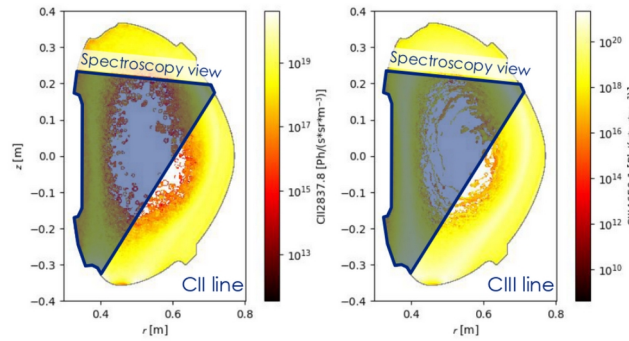
- **Simulations of the 2022 AUG experiment using HeatLMD code (IPP.CR) => 2 publications:**
 - J.G.A. Scholte et al., NME **37** (2023) 101522.
 - J. Cecrdle et al., Fusion Eng. and Design **194** (2023) 113886.
- **J. Cecrdle [PhD student]:**
 - experiments on Nano-PSI (DIFFER) to quantify the (so far unknown) tin (thermally enhanced) sputtering yield.
 - HeatLMD simulations of ITER-like $15 \text{ MJ/m}_{\parallel}^2$ unmitigated ELMs presented at ISFNT15.
 - parallelization of HeatLMD code in progress [**HeatLMD** = IPP.CR in-house code developed by J. Horacek for plasma-LM interactions].
- **S. Lukes [PhD student]:**
 - learning ERO2.0 from experts in FZJ (J. Romazanov, A. Kirshner, Ch. Baumann, S. Rode). Using the resources of the Karolina IT4I supercomputer.
 - implementation of Li into the code and first results (next slide).
 - results presented at the ITER summer school and at the IT4I user's conference.
- **In collaboration with Charles University:**
 - investigation of Sn Removal for Liquid Metal Tokamak Divertor by Low Pressure Argon Arc with Hot Tungsten Cathode System
 - published in Journal of Fusion Energy **42** 36 (2023) but no national funding for continuation.

HeatLMD + COREDIV integrated modeling for COMPASS-U (see I. Stanik's talk)

- COMPASS-U H-mode discharge simulated by Fiesta+METIS (half field scenario = 2.5 T, 0.8 MA)
- Plasma-LMD interactions by HeatLMD coupled to COREDIV = self-consistent 1D radial transport of impurities to plasma core & 2D multi-fluid transport in the SOL
- Li and Sn LM were considered yielding core concentrations:
 - $C^{Li} < 10\text{-}11\%$, corresponding to $P^{rad,Li} < 0.13 \text{ MW}$
 - $C^{Sn} < 0.025\%$, corresponding to $P^{rad,Sn} < 1.1 \text{ MW}$
- Article by Irena Stanik submitted to PoP (see her presentation for more details)

First ERO2.0 simulations of Li (& W) redeposition in COMPASS-U

- Verification on COMPASS experimental data
 - spectroscopy of carbon CII & CIII lines
 - 2019 LMD experiment Li redeposition on 14 screws
- Predictions for COMPASS-U
 - background plasma from SOLPS-ITER, H-mode scenario at 4.3 T, 1.2 MA
 - model ready to use, simulations on-going



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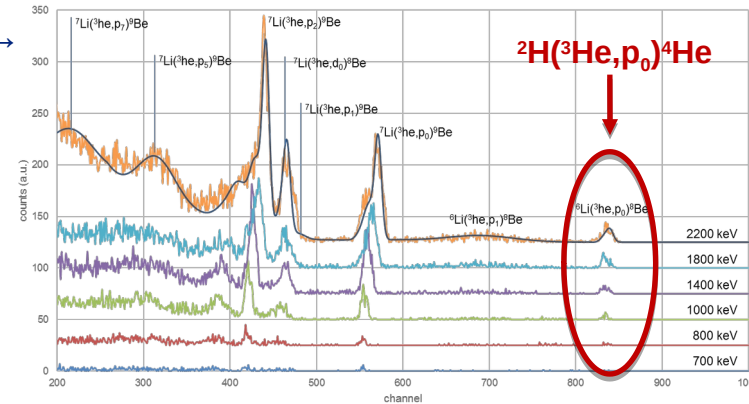
COMPASS-U LMD tile design plans for 2024

- **Design**
 - Modification of the present design according the outcomes of the workshop
 - Integration of the LMD tile design into the COMPASS-U divertor
 - Structural analysis from EM loads (W tile + CPS block)

Follow-up from 2019 COMPASS LMD experiments

- **D retention in Li and LiSn alloy (75% Sn) during ELMy H-mode exposures in COMPASS divertor**

- NRA analysis at **IPFN, Portugal** [R. Mateus, N. Catarino]
- using IPFN JET chamber & 2300 keV $^3\text{He}^+$ beam
- competing ^2H , ^6Li & ^7Li NRA emissions →

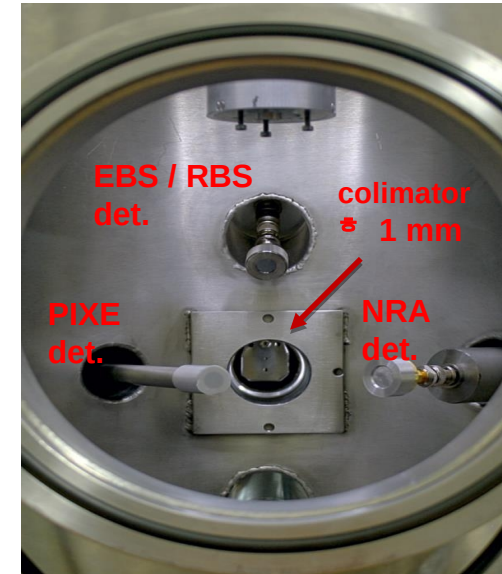


- **Exposure of LiSn module in OLMAT**

- experiment TBP after NRA analysis
- main goal = behavior of LiSn under HHF
 - Quantify erosion
 - Stability of LM
 - Vapor cooling effect
 - Change in Li depletion



JET chamber / Detector geometry



R. Mateus et al., Nucl. Instrum. Methods Phys. Res. B 486 (2021) 55

Modeling plans for 2024

- **Participation in OLMAT LiSn experiment + associated HeatLMD modeling:** J. Horacek, J. Cecrdle, A. de Castro
- **Coupled HeatLMD / COREDIV simulations of (Fiesta+METIS) COMPASS-U discharges:** J. Horacek, I. Stanik
 - preliminary results showed that a more rigorous assessment is needed by including into the workflow the following:
 - SOLPS-ITER (for divertor background);
 - ERO2.0 for the impurities (Li, Sn) transport inside LCFS and their deposition study assuming Marconi-Fusion cluster resources;
 - BIT1 (D. Tskhakaya's PIC code) for better prompt redeposition;
 - Possibly also ASTRA modeling of the central plasma Sn removal by 1 MW ECRH.
- **Coupling HeatLMD with a vapor cooling power model**
 - work in collaboration with G. Nallo from PoliTo
 - TBP after parallelization of HeatLMD