



Paper Rehearsal:

Modelling resistive-inductive evolution of currents in Wendelstein 7-X

Nucl. Fusion

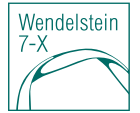


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Abstract



- The THRIFT code, capable of modelling toroidal current evolution in fusion plasmas, has been modernised and integrated into STELLOPT
- Several W7-X programs have been recreated using THRIFT and BOOTSJ, showing:
 - THRIFT characteristic timescales **agree** with experimental observations
 - BOOTSJ bootstrap current predictions **disagree** with experimental observations
 - Numerical issues still remain in THRIFT regarding edge plasma temperature
- ECCD and heating step simulations show THRIFT is capable of responding to changes in current sources

Rotational transform and the island divertor in W7-X

Currents affect the rotational transform: [Strand, P.I. & Houlberg, W.A. 2001 *Phys. Plasmas* **8** 2782]

$$\iota = \frac{\mu_0}{S_{11}} I - \frac{S_{12}}{S_{11}} \Phi$$

μ_0 – vacuum permeability
 I – toroidal current
 Φ – toroidal magnetic flux
 S_{ij} –susceptance matrix elements

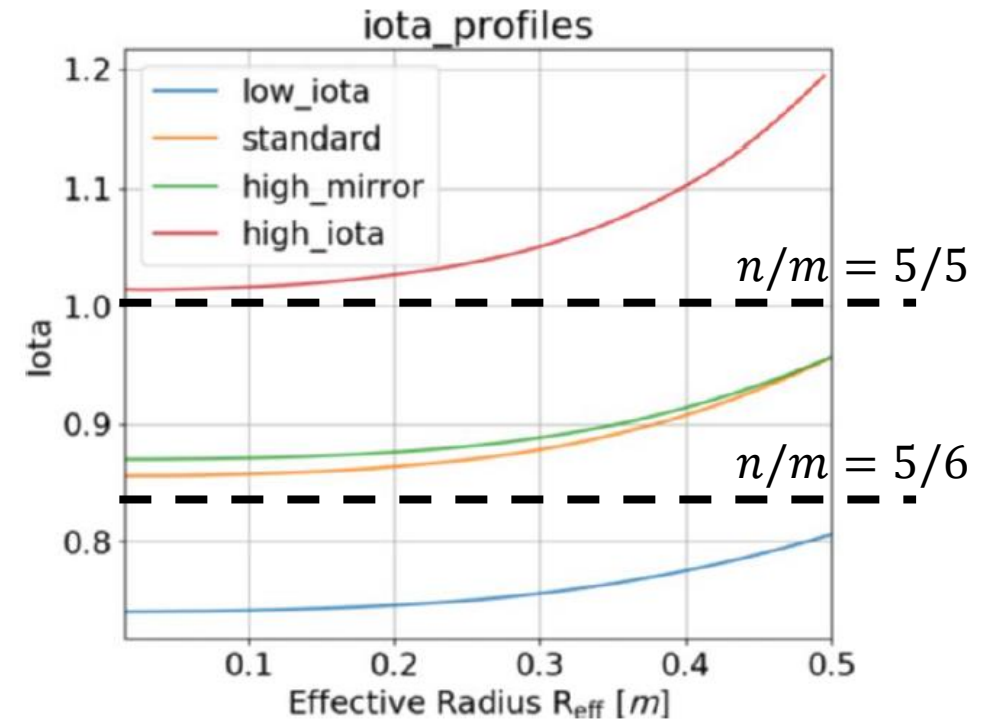
Effect on island divertor: [Yu Gao et al 2019 *Nucl. Fusion* **59** 106015]

- Shifting of strike-lines ($\sim 9\text{mm/kA}$, standard)
- Entering of limited configuration

Current sources:

- Bootstrap current
- Externally driven currents (ECCD, NBCD)

➤ Need for accurate source current models



Pedersen, T. et al. *Plasma Phys. Control Fusion* **61** (2019)



Currents evolve on inductive-resistive timescales

Total current evolves with $\tau_{L/R}$ ($\sim 30\text{s}$ in W7-X) $\rightarrow > 100\text{s}$ before saturation is reached

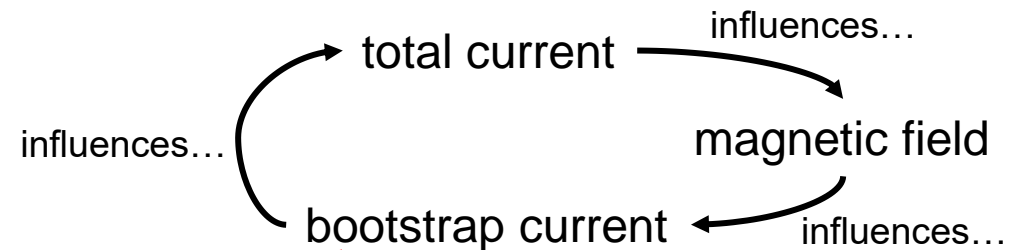
Mechanism:

1. $\nabla \times \mathbf{B} = \mu_0 \mathbf{J}$ toroidal current changes poloidal \mathbf{B} -component
2. $\nabla \times \mathbf{E} = -\partial \mathbf{B} / \partial t$ parallel electric field is induced
3. $J_{\text{shield}} = \sigma_{\parallel} E_{\parallel}$ “shielding” current opposes total current

Modelled with simple fit: $I(t) = I_{\infty} \left(1 - \exp\left(-\frac{t}{\tau_{L/R}}\right) \right)$

Downsides:

- $I_{\infty}, \tau_{L/R}$ not necessarily time-independent
- Only gives total current, not the distribution
- Doesn't include effect of magnetic field



➤ **THRIFT self-consistently models this process**

The THRIFT code (1): System of equations

Diffusion equation for ι : [Strand, P.I. & Houlberg, W.A. 2001 *Phys. Plasmas* **8** 2782]

$$\frac{d\iota}{dt} = \frac{d\iota}{d\Phi} \frac{d\Phi}{dt} + \frac{d}{d\Phi} \left[\frac{\eta_{\parallel}}{\mu_0} \Phi' (S_{11}\iota + S_{12})^2 \frac{d}{d\rho} \left(\frac{S_{11}\iota + S_{12}}{S_{21}\iota + S_{22}} \right) - \eta_{\parallel} \langle J_s B \rangle \frac{dV}{d\Phi} \right]$$

ρ – radial variable
 J_s – net source current
 V – plasma volume

$$u = S_{11}\iota + S_{12} = \frac{\mu_0}{\Phi_a} I$$

[Schmitt, J.C. PhD thesis (2011)]

$$\frac{du}{dt} = \frac{S_{11}}{\Phi_a^2} \frac{d}{ds} \left[\eta_{\parallel} V' \left(\frac{\langle B^2 \rangle}{\mu_0} u' + p' u - \langle J_s B \rangle \right) \right]$$

$s = \Phi/\Phi_a$
 p – pressure

$$\frac{du}{dt} = \alpha_1 + \alpha_2 u + \alpha_3 u' + \alpha_4 u''$$

$$\alpha_1 = -\frac{S_{11}}{\Phi_a^2} \frac{d}{ds} [\eta_{\parallel} V' \langle J_s B \rangle]$$

$$\alpha_2 = \frac{S_{11}}{\Phi_a^2} \frac{d}{ds} [\eta_{\parallel} V' p']$$

$$\alpha_3 = \frac{S_{11}}{\Phi_a^2} \left(\frac{d}{ds} \left[\eta_{\parallel} V' \frac{\langle B^2 \rangle}{\mu_0} \right] + \eta_{\parallel} V' p' \right)$$

$$\alpha_4 = \frac{S_{11}}{\Phi_a^2} \eta_{\parallel} V' \frac{\langle B^2 \rangle}{\mu_0}$$

❖ Discretise radial $s = \Phi/\Phi_a$ -grid (ns points) & apply Backwards Time, Centred Space (BTCS) scheme

Boundary conditions:

- Magnetic axis ($s = 0$): No current enclosed $\rightarrow u = 0$
- Plasma edge ($s = 1$): Voltage in inductor $\rightarrow E_{\parallel} = -\frac{L_{\text{ext}}}{2\pi R} \frac{dI}{dt}$ L_{ext} – external inductance
 R – major radius

Initial condition: No currents anywhere $\rightarrow u = 0$

$T \rightarrow 0 \Rightarrow \eta_{\parallel} \rightarrow \infty$
 T clamped to $> 14\text{eV}$

Verifying the THRIFT implementation

Test case:

1. Large aspect ratio, circular cross-section tokamak
2. Spatially uniform resistivity
3. Time-independent source currents

$$\mu_0 \sigma_{\parallel} \frac{\partial E_{\parallel}}{\partial t} = \frac{1}{r} \frac{\partial}{\partial r} \left(r \frac{\partial E_{\parallel}}{\partial r} \right) \rightarrow$$

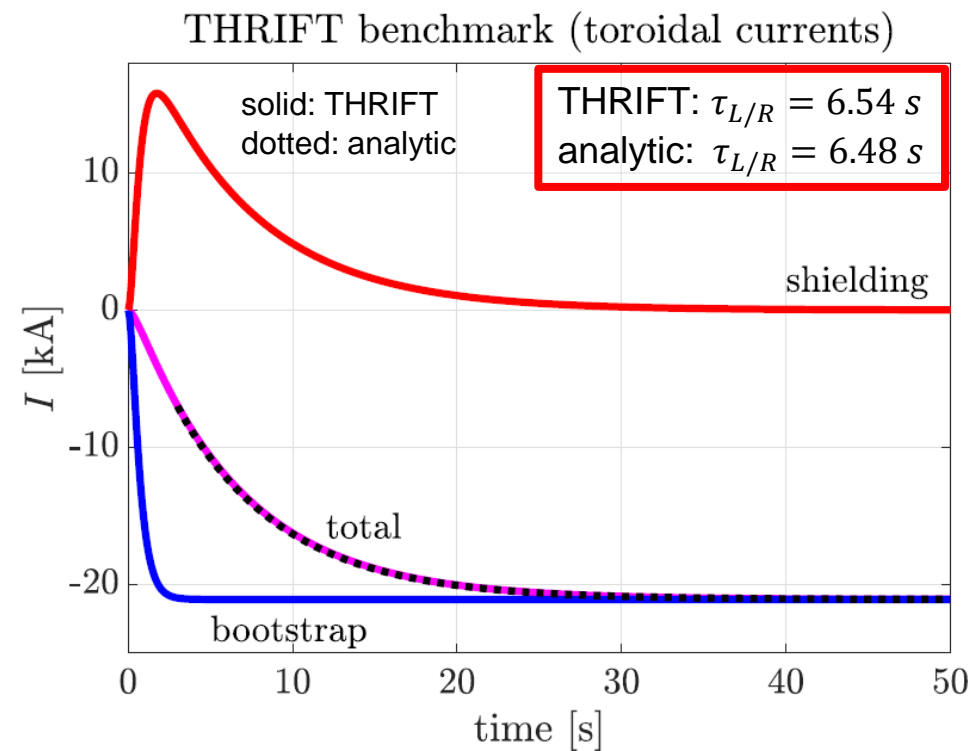
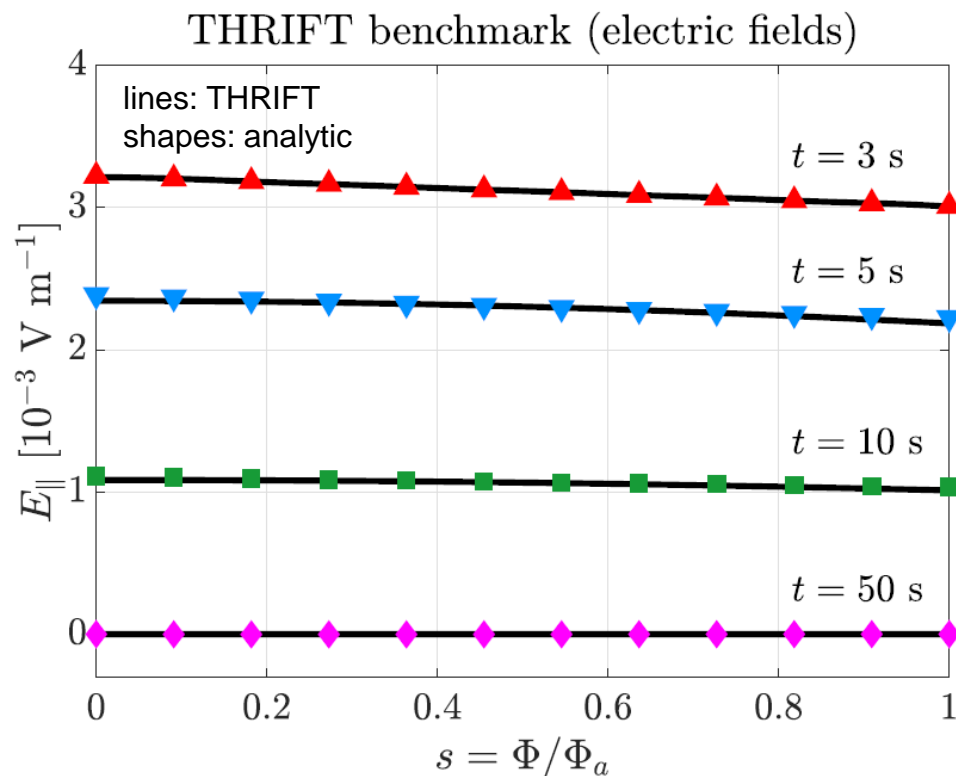
$$E^{FB}(x, t) = \sum_{n=1}^{\infty} f_n F_n(x) \exp\left(-\frac{t}{\tau_n}\right)$$

F_n - Bessel function
 f_n - amplitude
 τ_n - decay time

THRIFT case:

1. Generate VMEC equilibrium circular cross section, $R/a = 1000$ tokamak
2. η_{Sauter} independent of T_i \rightarrow Generate spatially uniform T_e, n_e, n_i profiles
3. Generate spatially varying T_i $\rightarrow \nabla T_i$ -driven bootstrap current $J_{bsc}(s) = \frac{\sqrt{\epsilon} R}{\Phi_a} p' s$
 - $I(s, t = 0) = 0 \rightarrow$ VMEC cannot sustain pressure gradient
 - Evolve n, T profiles to steady state values over ~ 3 seconds
 - Determine f_n, τ_n at $t = 3$ s from THRIFT data, then compare at later times

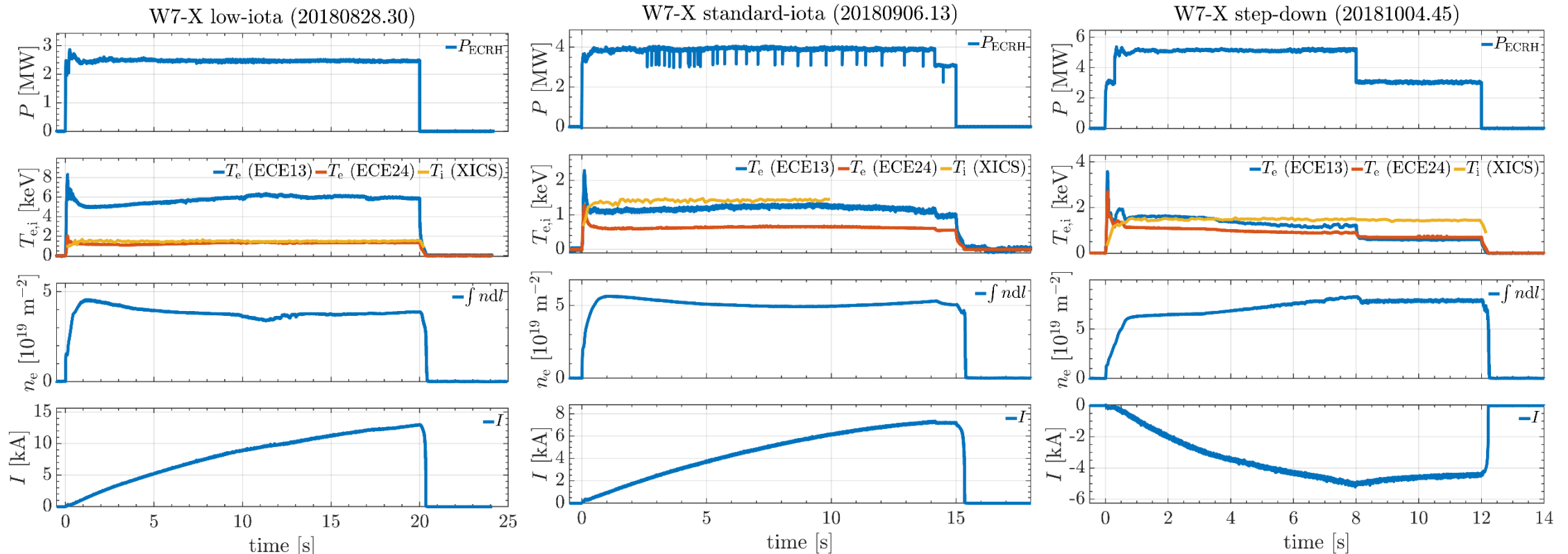
Verification results



- THRIFT electric field closely follows analytical field for $t > 3 \text{ s}$
- Decay times for THRIFT and analytical currents agree

Implementation successful

Current evolution simulations for W7-X



fixed heating, low-iota

fixed heating, standard

heating step-down,
low-iota, reverse field

Set-up in THRIFT:

- n, T profiles produced by WAPID_FIT
- η_{\parallel} calculated using Sauter model
- Bootstrap current calculated by BOOTSJ

Results for low-iota, fixed heating case (I)

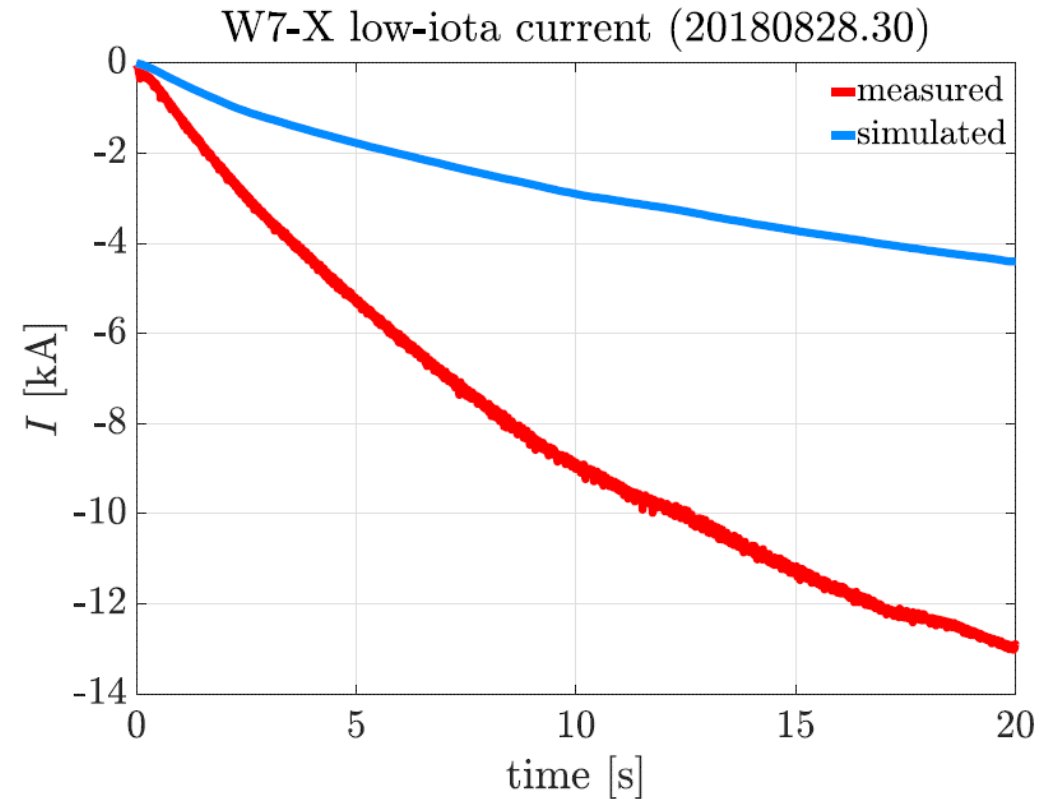
Curve fits:

$$I(t) = I_{\infty} (1 - \exp(-t/\tau_{L/R}))$$

	$\tau_{L/R}$ [s]	I_{∞} [kA]	I_{BS} [kA]
experiment	12.76 ± 0.36	16.32 ± 0.24	—
simulation	13.22 ± 0.33	5.53 ± 0.03	8.65

experimental uncertainties from fitting;
simulation uncertainties from profile uncertainties

- Quantitative agreement in $\tau_{L/R}$ (~4%)
- Disagreement in currents:
 - Bootstrap current ~half the experimental I_{∞}
 - Noticeable difference (few kA) between simulation I_{∞} and I_{BS}



➤ **BOOTSJ underestimates the bootstrap current**

Results for standard-iota, fixed heating case (II)

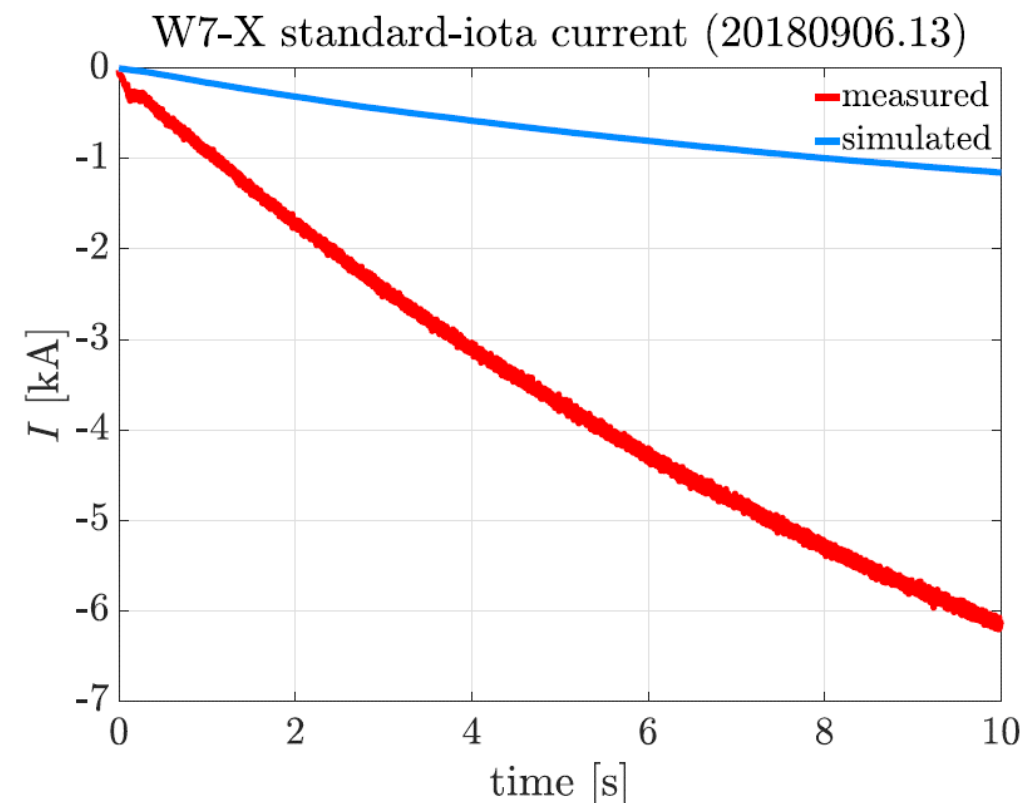
Curve fits:

$$I(t) = I_{\infty} (1 - \exp(-t/\tau_{L/R}))$$

	$\tau_{L/R}$ [s]	I_{∞} [kA]	I_{BS} [kA]
experiment	12.16 ± 0.07	5.53 ± 0.03	—
simulation	11.09 ± 0.003	1.91 ± 0.001	3.31

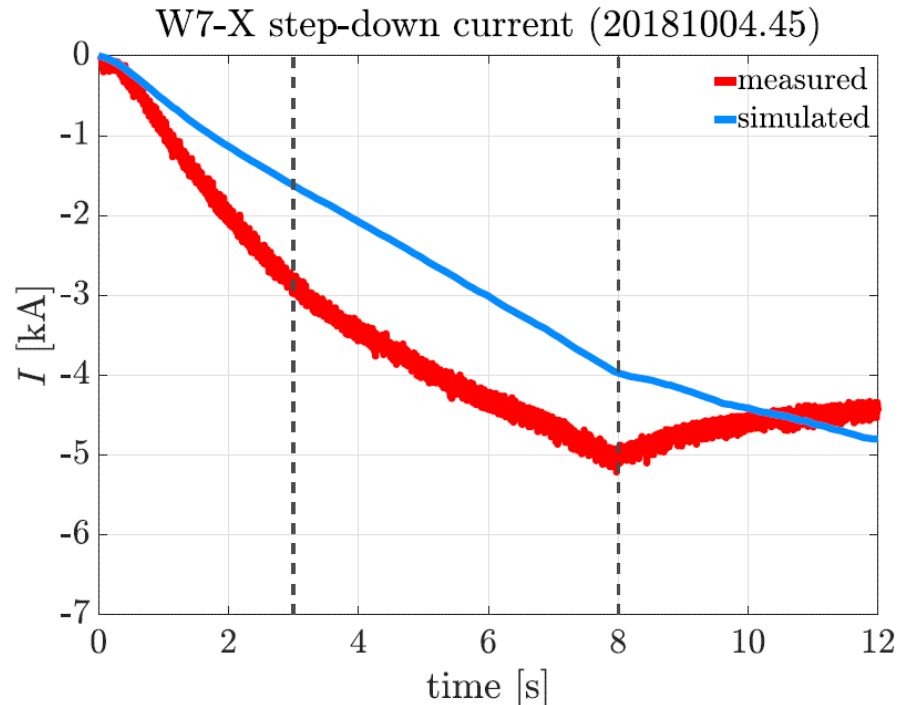
uncertainties from fitting

- Quantitative agreement in $\tau_{L/R}$ (~10%)
- Disagreement in currents:
 - Bootstrap current $\sim 0.6 \times$ experimental I_{∞}
 - Small difference between simulation I_{∞} and I_{BS}



➤ **BOOTSJ underestimates the bootstrap current**

Results for reverse-field, low-iota, heating step down case (III)

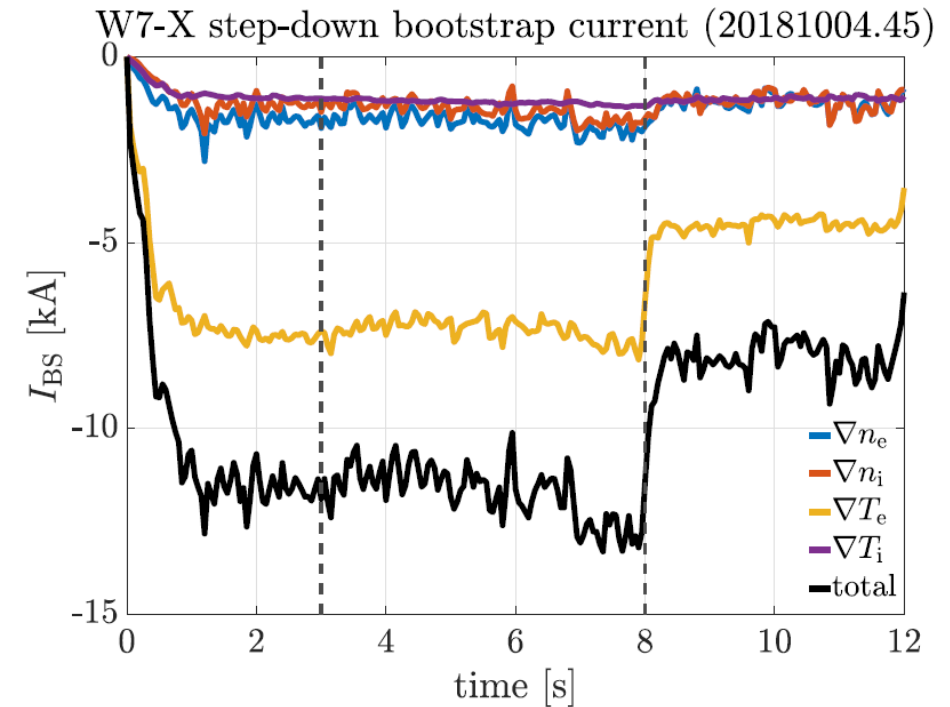


Before heating step:

- Disagreement in currents

After heating step:

- Disagreement in currents:
 - Experimental current decays
 - Simulated current continues growing



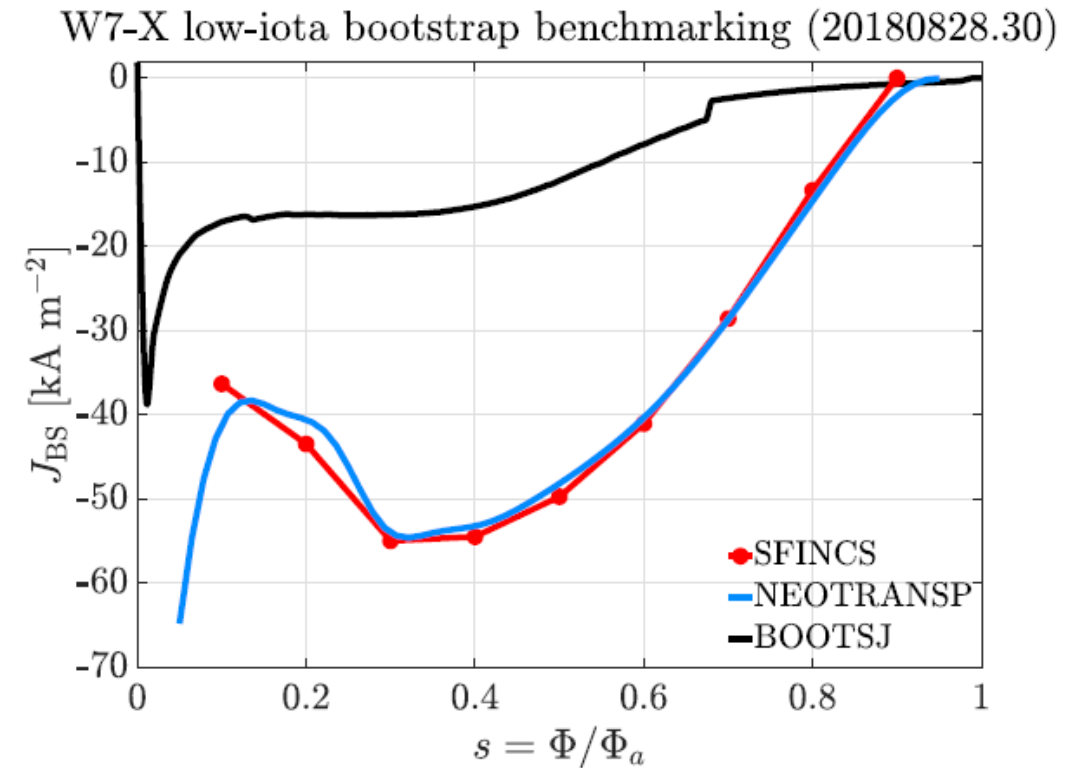
BOOTSJ bootstrap before step: ~ 12 kA

BOOTSJ bootstrap after step: ~ 8 kA

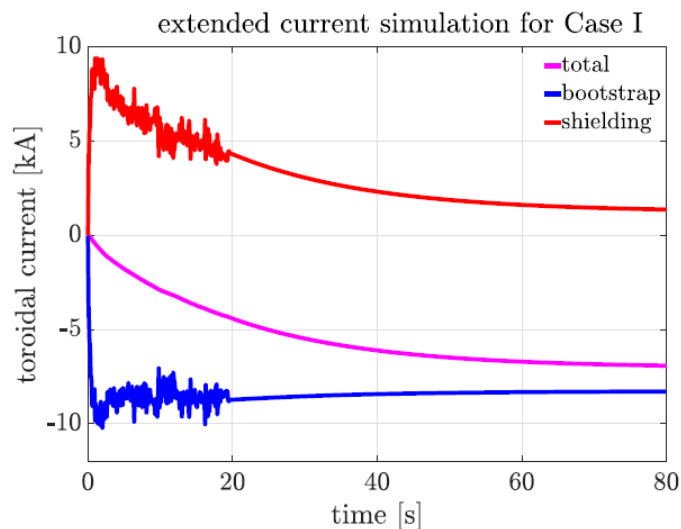
- **BOOTSJ overestimates** the bootstrap current

(I) Bootstrap current predictions by BOOTSJ

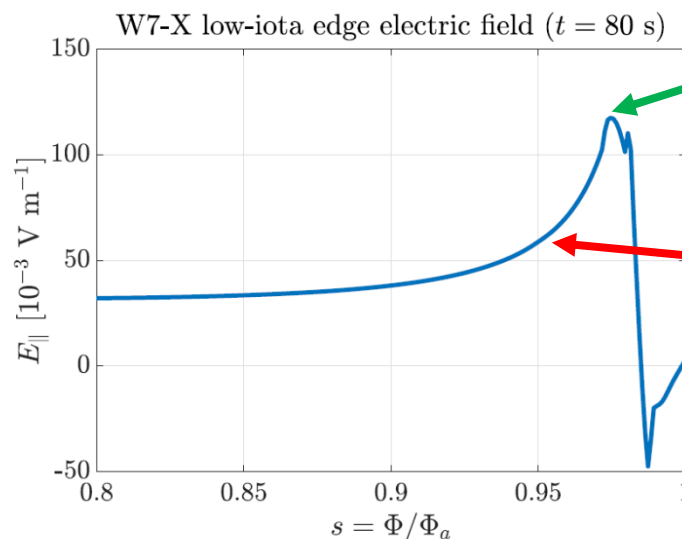
- BOOTSJ model is derived in the asymptotic collisionless limit
 - For this case, compare estimate of bootstrap current from BOOTSJ with:
 - SFINCS
 - NEOTRANSP
- BOOTSJ inaccurate at finite collisionalities



(I) Shielding current does not vanish in steady-state

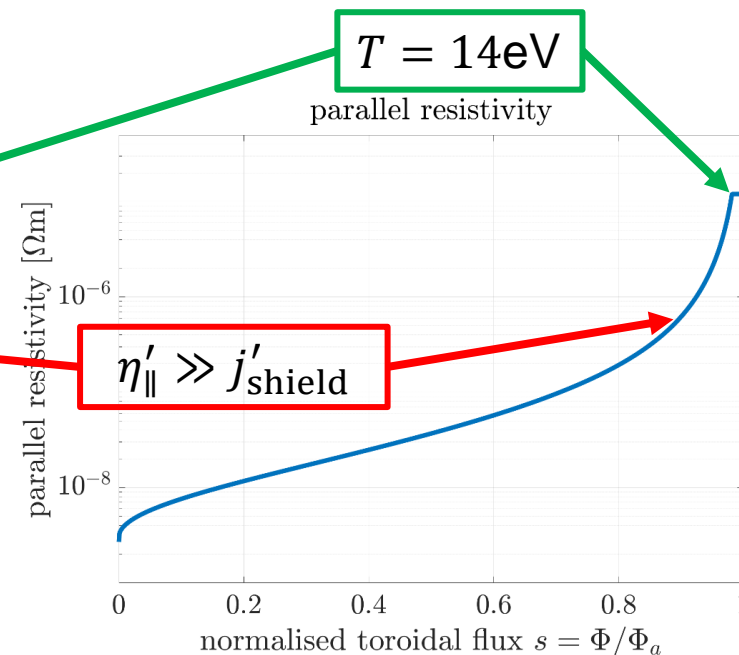


- Shielding current $\rightarrow 0$ in simulation



- Edge BC: $E_{||} = -\frac{L_{ext}}{2\pi R} \frac{dI}{dt} \rightarrow 0$
- Core electric field becomes “frozen-in” due to edge structure

$$E_{||} = \eta_{||} j_{shield}$$



- $\eta_{||}$ flattens when T clamped
- Small residual shielding current in the plasma edge

...but $\eta_{||}$ **increases** faster than j_{shield} **decreases**.

➤ Different edge temperature model/clamping ($\sim 100\text{eV}$) or a different resistivity model necessary

Current acceleration with ECCD

- Some divertor scenarios require certain $I = I_{BS} = I_{\text{target}}$

$$I(t) = I_{\infty} \left(1 - \exp(-t/\tau_{L/R}) \right)$$
$$\rightarrow \dot{I}(t) \propto I_{\infty} = I_{BS} + I_{\text{ext}}$$

- Temporarily drive current I_{ext} to reach $I = I_{\text{target}}$

Set-up in THRIFT:

- Add ECCD to low-iota, fixed heating case:

$$J_{\text{ECCD}}(\rho) = I_{\text{norm}} \exp\left(-\frac{(\rho - r_c)^2}{w^2}\right)$$

$$\rho = \sqrt{s} = \sqrt{\Phi/\Phi_a}$$

$r_c = 0.3$ –deposition location

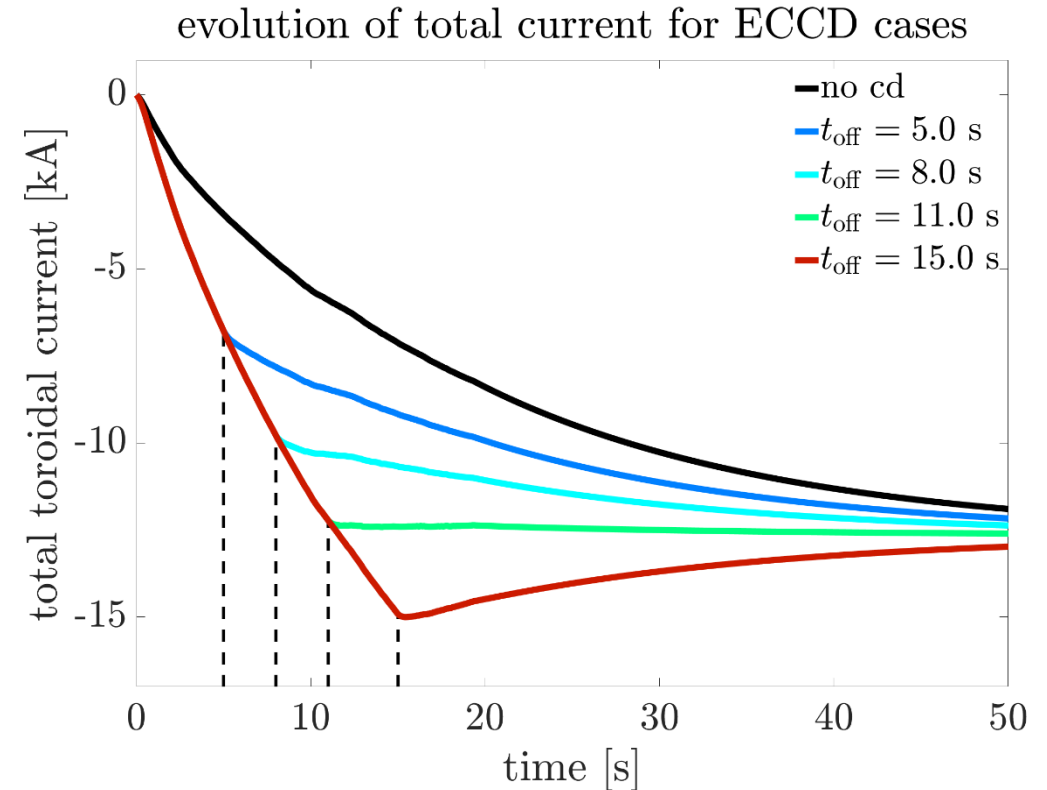
$w = 0.175$ –deposition width

[Yu Turkin et al, *Fusion Sci. Technol.* **50**, 387-394 (2006)]

- Apply $I_{\text{ECCD}} = 20$ kA for durations $t_{\text{ECCD}} = 5$ s, 8 s, 11 s, 15 s

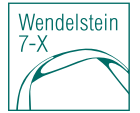
Current acceleration with THRIFT

- No ECCD: I_∞ in > 50 s
- ECCD with $t_{\text{off}} = 5$ s \rightarrow undershoot I_∞
- ECCD with $t_{\text{off}} = 8$ s \rightarrow undershoot I_∞
- ECCD with $t_{\text{off}} = 11$ s $\rightarrow I_\infty$
- ECCD with $t_{\text{off}} = 15$ s \rightarrow overshoot I_∞



- THRIFT captures changes in source currents
- CD scenario investigation possible with THRIFT:
 - Amount of CD necessary to reach target currents
 - Effect of CD on iota profiles

Conclusion

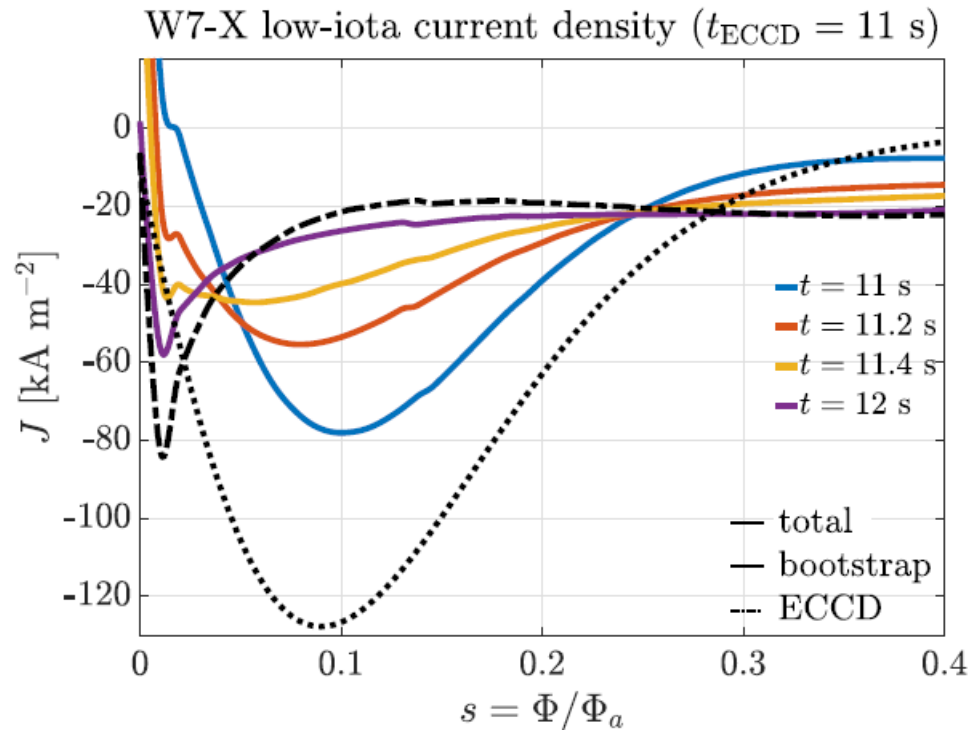


- Reimplemented and verified the THRIFT code
- Decay times of plasma currents in W7-X are recreated by THRIFT
- BOOTSJ predictions inaccurate → Plasma collisionality
- THRIFT shielding current does not vanish → Different treatment of edge T necessary
- Changes in source currents captured by THRIFT

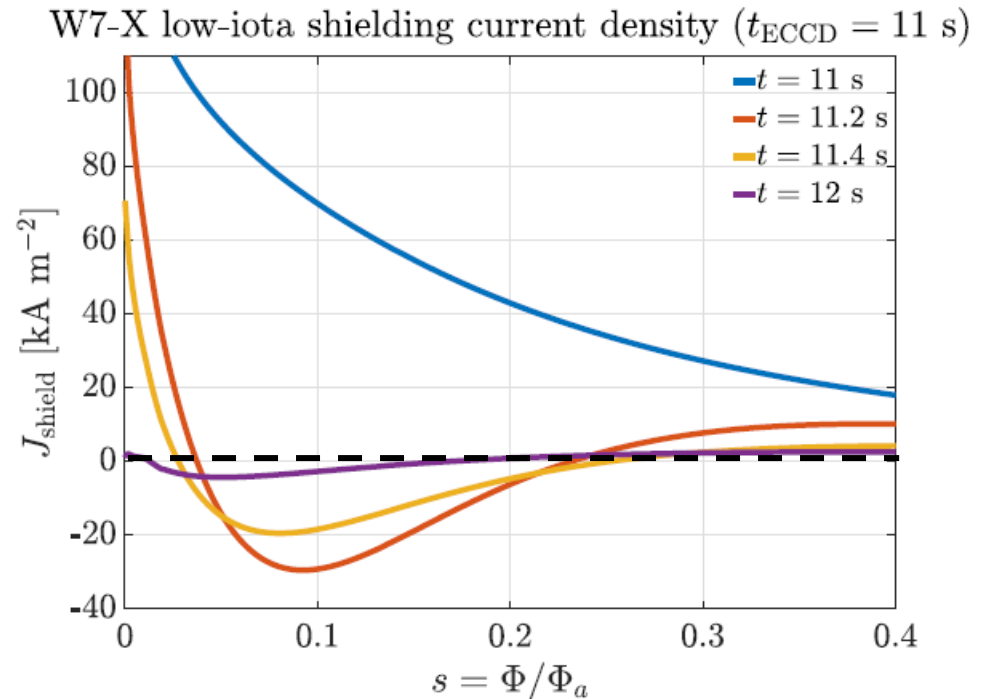
Future potential uses of THRIFT:

- Validating/benchmarking other source current codes
- Investigating current drive scenarios

Current density for $t_{\text{ECCD}} = 11 \text{ s}$

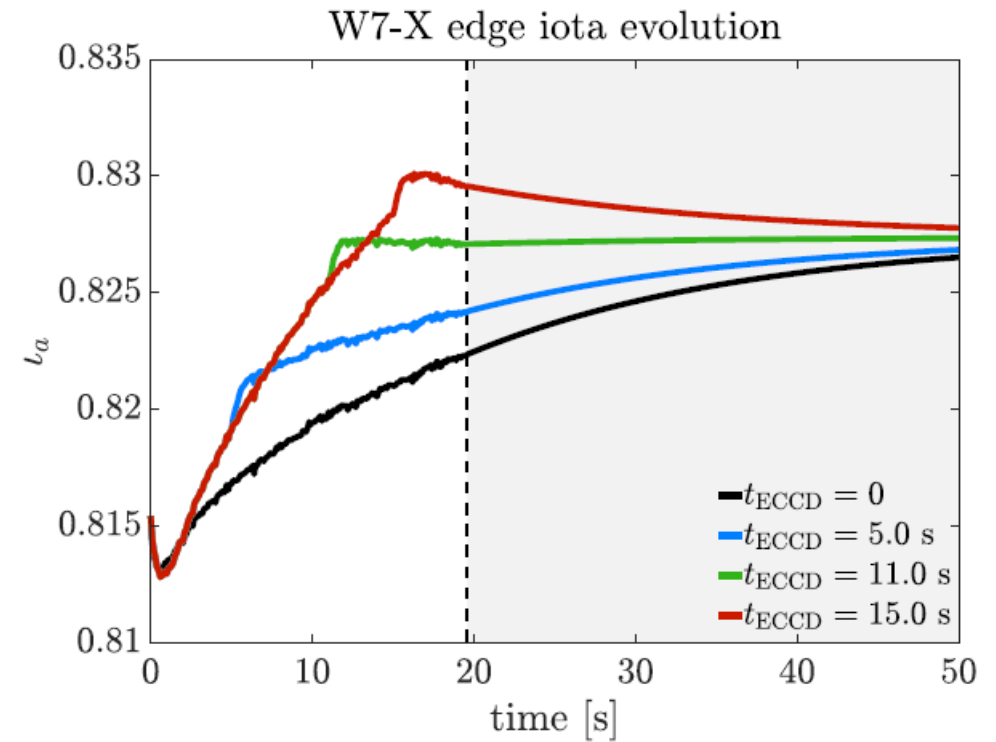
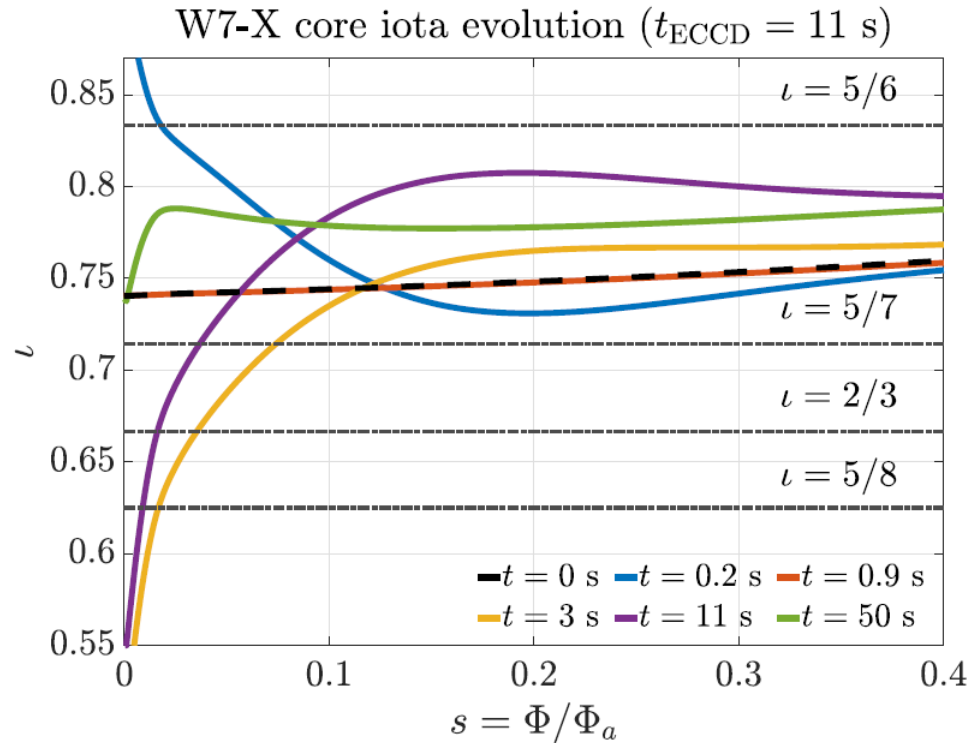


- Total current remains unchanged
- Total current density redistributes to match bootstrap current
- Redistribution occurs on timescale of $\sim 1 \text{ s}$



- Shielding current $\propto E_{\parallel}$
- E_{\parallel} becomes locally negative (in direction of total current) \rightarrow rapid diffusion from “negative” to “positive” locations
- $\int E_{\parallel} ds = 0$

Iota evolution with ECCD

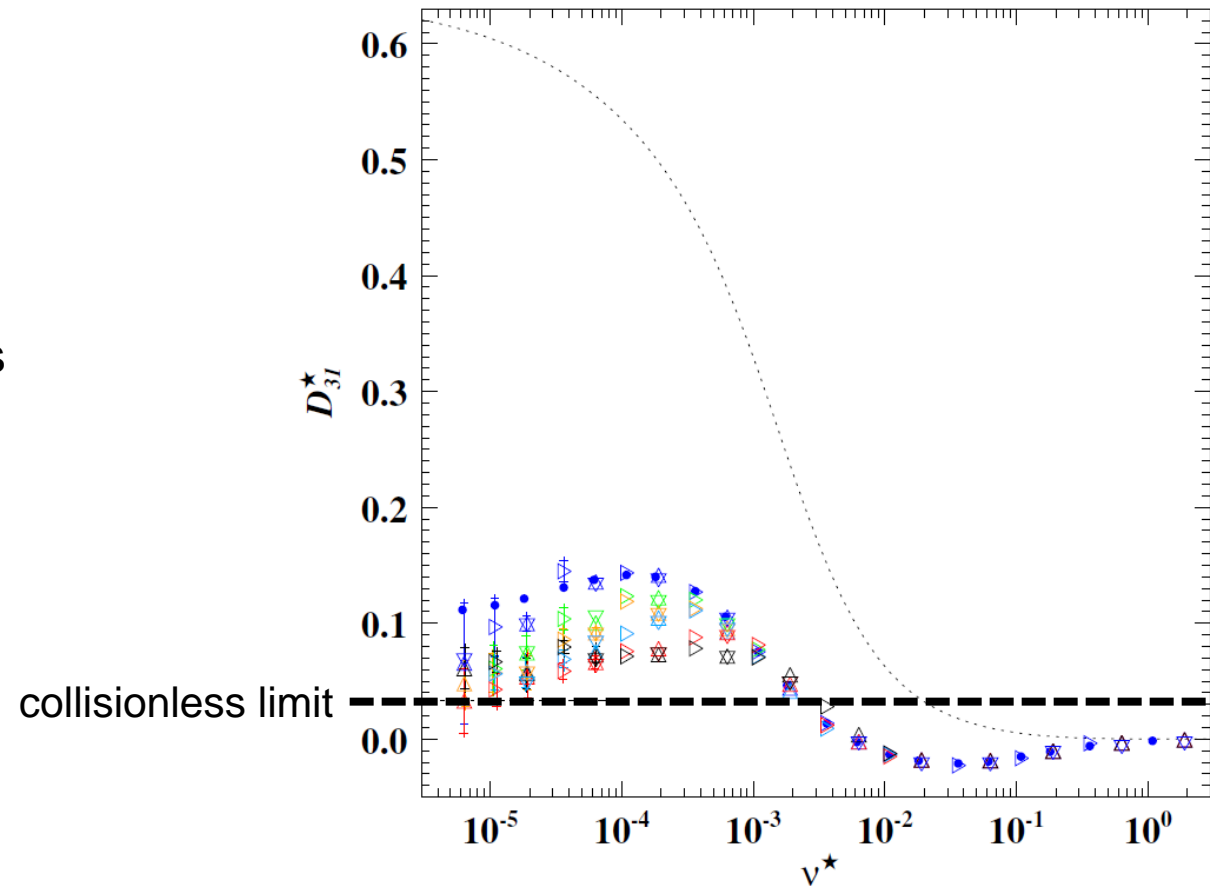


Mono-energetic bootstrap current coefficient for W7-X



BOOTSJ underpredicts at small
(finite) collisionalities, overpredicts
at large collisionalities

mono-energetic bootstrap current coefficient (standard-iota)



Beidler C.D. et al. *Nucl. Fusion* **51** (2011) 076001