

PSD Project Board Meeting 30.10.2024

For the W7-X team M.Jakubowski (TFL), A.Alonso(DTFL), I.Calvo (DTFL), J.Haese(PSO)



This work has been carried out within the framework of the EUROfusion Consortium, funded by the European Union via the Euratom Research and Training Programme (Grant Agreement No 101052200 — EUROfusion). Views and opinions expressed are however those of the author(s) only and do not necessarily reflect those of the European Union or the European Commission. Neither the European Union nor the European Commission can be held responsible for them.



High-level goals of WP W7-X

WPW7X: Focuses on contributing to and leveraging W7-X to demonstrate physics questions related to HELIAS line towards fusion reactor:

- The positive effects of optimization on plasma confinement, fast particle behavior, and MHD equilibrium and stability.
- Achieving good plasma confinement in the long-mean-free-path regime at elevated plasma beta.
- Ensuring safe steady-state operation while exploring potential reactor scenarios.

For WPW7X, 2025 is an as-planned continuation of the FP9 strategy. The plan is adapted to technical capabilities, findings and achievements (e.g. 1.3 GJ energy turn-around)

- Main objective 2025: conduction of campaign W7-X (heating upgrades, pellet injection, low field operation, wall conditioning/metallic wall, other RTs)
- > EU contributions to upgrades in FP9
- Exploitation of 2024/25 campaign
- Physics basis & ITER support



Research topics for experimental campaigns 2024/2025

ID	Tag	Description
RT-01	High performance conditions	Exploration of reduced turbulence/ high-performance
		scenarios in view of stationary plasma conditions with
		temperature-, density and impurity-profile control.
RT-02	Heating scenarios	Exploration of heating scenarios using upgraded heating
		capabilities (ECRH, NBI, ICRH).
RT-03	High beta scenario	Development of high plasma beta scenario by low field
	development	operation.
RT-04	Long-pulse operation and wall	Development of integrated scenarios for long-pulse operation
	conditioning	with PFC heat-load control, efficient particle exhaust and
		impurity screening; Development of wall conditioning
		procedures.
RT-05	Detachment	Development of long and stationary divertor detachment
		scenarios with and without impurity seeding.



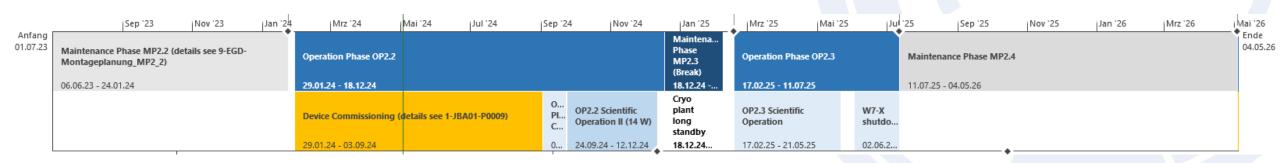
Research topics for experimental campaigns 2024/2025

ID	Tag	Description
RT-06	Tungsten PFCs (together with WP PWIE)	Exploration of scenarios compatible with carbon free operation and tungsten PFCs.
RT-07	Documentation of physics basis	Physics basis (core, edge) and reference discharges.
RT-08	Core physics studies	Completion of the core transport and stability physics basis in the extended operational space.
RT-09	Edge physics studies	Completion of the edge and SOL physics basis in the magnetic configuration space of W7-X.



Status of Wendelstein 7-X

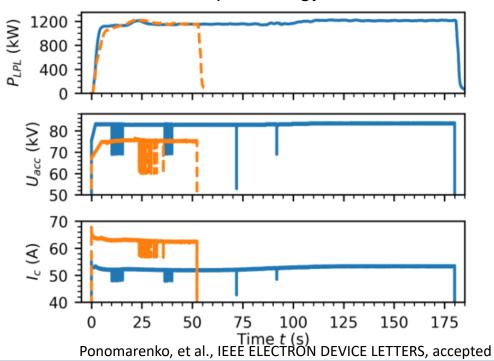
- Technical commissioning completed sucesfully ✓
- Plasma commissioning started 3 Sep 2024, ✓
- Plasma scientific operation OP2.2 01 Oct 2024 until 12 Dec 2024
- Plasma scientific operation OP2.3 17 Feb 2025 until 21 May 2025
- Program Workshop took place on 24-24 Apr 2024
- Campaign participation in OP2.2 assigned ✓
- Campaign participation in OP2.3 preliminary assigned. Announcement Nov 2024.





ECRH upgrades

- 1.5 MW successfully installed and commissioned for OP2.2
- ⇒ The 1-ms short-pulse tests confirmed the nominal output power of 1.5 MW
- ⇒At 1.3 MW pulse lengths of 3 minutes achieved
- ⇒At 1.2 MW pulse length of 580 s demonstrated
- ⇒Thales will build 3 further 1.5 MW
- Contract to develop 2 MW gyrotron awarded to Kyoto Engineering



ID	Tag
RT-01	High performance conditions
RT-02	Heating scenarios
RT-04	Long-pulse operation and wall conditioning
RT-05	Detachment



Enhancements to heating systems

- NBI 55 kV H injection with all 4 sources S3, S4, S7, S8
 - injected power approx. 2-2.2 MW
 - pulse length: max. 5 sec integral
- NBI 42 kV ⁴He injection with 4 sources S3, S4, S7, S8
 - injected power approx. 1.2 MW (tbc)
 - pulse length 5 sec (tbc)
- ICRH (with FZJ, ERM/KMS)
 - Power > 1 MW (400 kW in comissioning)
 - pulse length up to 10 s
 - Scenarios:
 - H minority in He plasmas
 - ³He minority heating in ⁴He or H
 - 3-ion-heating with ³He, H, ⁴He
 - Plasma start-up (at 1.7 T)



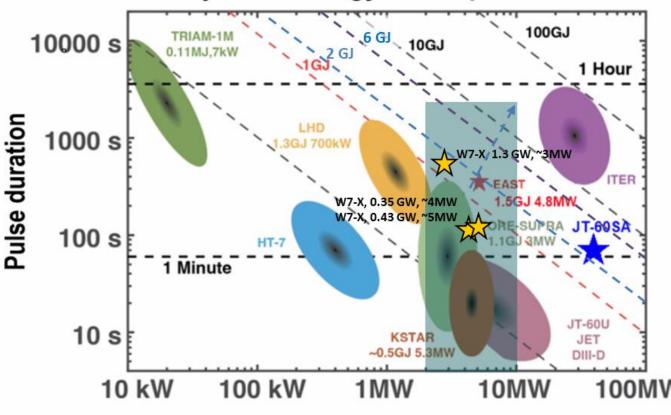
100	Particulary important for			
	ID	Tag		
	RT-01	High performance conditions		
	RT-02	Heating scenarios		
	RT-04	Long-pulse operation and wall conditioning		
	RT-08	Core physics studies		



Development of long pulse

- In OP2.2/2.3 the next milestone of LPO will be reached
 - 2 GJ attached discharge:
 - test scenarios to accomodate low plasma current
 - Operate at power fluxes close to reactor design point
 Monitor impurities in steady-state
 - (incl. tungsten)
 - 2 GJ detached, if feedback on plasma radiation will be implemented.
 - Based on mixture of intrinsic and seeded impurities
 - Move to higher P_{SOL}
 - Additionally scenarios with high performance plasmas will be developed to prolong their duration, e.g. with steady-state pellet injector/NBI.

Injected energy in the plasma



Injected input power in the plasma

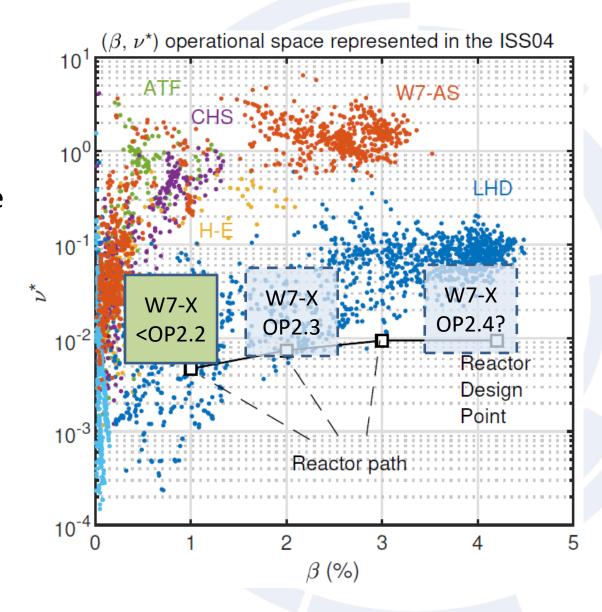
time [s]



Importance of developing low-B scenarios at W7-X

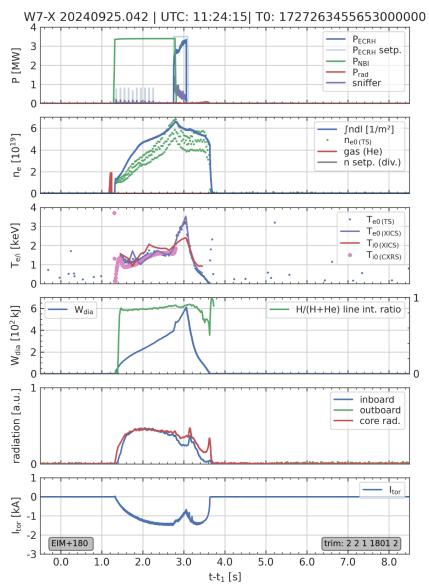
The development of low-B scenarios at W7-X (namely, 1.7 T for X3-ECH heating) is programatically important beacause:

- 1. Plasma start-up not based on X2-ECH will be required in a reactor. At W7-X, also the combination of ICH and NBI is being pursued.
- 2. Low-B operation at W7-X allows an easier access to high- β , low- ν * operation, potentially close to reactor design values (see next slide).
- 3. Investigate B transport dependencies for reactor projections. Notably, there is no stellarator scaling of the SOL e-folding length.





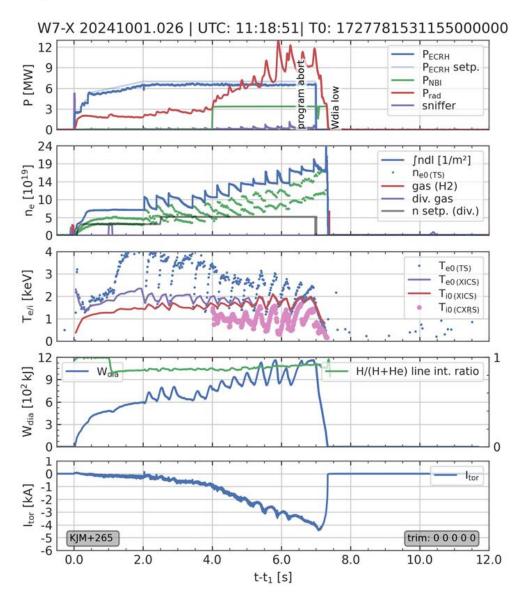
Plasma start-up at 1.8 T with ECRH & ICRH



- A full plasma start up in the core could be achieved with a combination of X2 ECRH (101GHz, 250 kW, 40ms duration) and NBI (2 sources).
- IRCH was sucesfully taken into operation. Power deposition within the SOL island and at the separatrix.
 Further optimization required.
- Operation at lower field provides easier route towards high beta plasma → closer to reactor design point.



Steady-state pellet injector goes into operation OP2.2



pellet material

pellet size

pellet speed

repetition frequency

injection duration

injection modes

H₂ or D₂

2mm – 3mm (adjustable)

250 – 1000 m/s

single on demand, continuous up to 10 Hz

up to 30min tested

feed-forward, density feedback control foreseen beyond OP 2.3



Deliverables and milestones for 2024/2025

with fast-ions and transport regimes for long steady-state high- beta operation (energy limit 6 GJ) - modification requested W7X.D.08 D03.08 Deliverable Report on conducted Scenario & campaign preparation (focus: turbulent and neoclassical transport, high-power steady-state	31.12.2024 31.12.2024
turbulent and neoclassical transport, high-power steady-state	31.12.2024
operation) – on track	
W7X.D.09 D03.09 Deliverable Assessment report on scenarios with optimized transport and high-beta operation (energy limit 6 GJ) – modification requested	31.12.2024
W7X.D.10 Deliverable Verified and validated stellarator gyrokinetic codes for the calculation of turbulent transport (TSVV-13) – on track	31.12.2025
W7X.D.11 D03.11 Deliverable Report on conducted scenario & campaign preparation (focus: high-power steady-state operation) – on track	31.12.2025
W7X.D.12 Deliverable Assessment report on HELIAS optimization (with data from carbon PFC operation) (energy limit 18 GJ) – modification requested	31.12.2025
W7X.D.13 Deliverable Report on conducted scenario & campaign preparation (focus: PFC upgrades) – on track	31.12.2025
W7X.D.14 D03.14 Deliverable Comparative assessment of the HELIAS reactor physics basis with respect to other stellarator concepts (with International Collaborations). — on track	31.12.2025



Change of deliverabls and milestones

FSD	W7X	D3.9	W7X.D.09	D03.09	Deliverable	Assessment report on scenarion (energy limit 6 GJ)	os with optimized tra	ansport and high-beta operation	31.12.2024	48
hould	be									
FSD 3.	W7X	D3.9	W7X.D.09	D03.09	Deliverable	Assessment report on scenarios with optimized transport and high-beta operation (energy limit 2 GJ)	31.12.2024	48		on track
FSD	W7X	M20	W7X.M.06	M03.06	Milestone	Operation with High power and long-pulse Completed and 6 GJ energy turn-around achieved (pulse lengths up to 600 s, long-pulse detachment).	31.12.2025	60		Unknown
Should	d be									
FSD	W7X	M20	W7X.M.06	M03.06	Milestone	Operation with High power and long-pulse Completed and 2 GJ energy turn-around achieved (pulse lengths up to 600 s, long-pulse detachment).	31.12.2025	60		Unknown
FSD	W7X	D3.12	W7X.D.12	D03.12	Deliverable	Assessment report on HELIAS optimization (with data from carbon PFC operation) (energy limit 18 GJ)	31.12.2025	60		Unknown
Should	d be									
FSD	W7X	D3.12	W7X.D.12	D03.12	Deliverable	Assessment report on HELIAS optimization (with data from carbon PFC operation) (energy limit 18 GJ)	31.12.2025	60		Unknown



ID	Proposal description	Justification	PM @ 50%	BEN	CC [k€]
W7X-1	Complete the HELIAS physics basis and develop high performance scenarios for W7-X - validation of physics models and codes (fast ions, turbulence characterization, enhanced confinement regimes)	Enhance the involvement of the CIEMAT team in studies related to HELIAS physics to expedite progress and ensure timely readiness for the decision point regarding the HELIAS-based reactor.	24	CIEMAT	81.80
	Increase the efforts on the analysis and modelling for OP2.2 & OP2.3:		42		112.66
	- EMC3-Eirene modelling of high β plasmas, incl. synthetic diagnostics, AI to provide quick assessments - analysis of divertor plasmas with four endoscopes	Increased expenditures driven by the emphasis on 'high beta' and 'long	12	FZJ	56.79
W7X-2	 Validation of PHA and C/O monitor experimental data for long pulses using modelling; Development of scripts for fast data processing; 	pulse' research in OP2.3, aimed at studying divertor physics in high-beta discharges, as well as the behavior of impurities and filaments at the plasma boundary.	12	IPPLM	23.81
	 fast cameras data: fueling (pellets), high performance, impurity transport (TESPEL) filaments at the plasma boundary and alkali beam: density profiles, CXRS 		18	EK-CER	32.06



ID	Proposal description	Justification	PM @ 50%	BEN	CC [k€]
W7X-3	Insufficient missions budget in 2025. Funding source: unused 2024 secondment budget of 63k€ (not allocated)	Bringing mission budget to the level of 2024 is essential; otherwise, participation in the campaign could prove inadequate, significantly increasing the risk of not achieving the 2025 objectives and overall deliverables.			63 k€



Evaluate critical physics uncertainties associated with a Stellarator DEMO

- **Timeline:** Establish high-level objectives by June 2025, followed by an in-depth analysis to be concluded by the end of 2025.
- **Approach:** The assessment will concentrate on six key areas, with each area led by two experts specializing in the respective field of focus.
 - Heat and particle exhaust in the island divertor
 - MHD equilibrium and stability
 - Core transport and confinement
 - Fast particle confinement and interaction with Alfvén waves
 - Plasma-wall interaction
 - Scenario integration
- **Deliverables:** A document with a summary of high-level objectives will be completed by the end of June 2025, followed by a comprehensive analysis report, including references to relevant publications, to be finalized by the end of 2025.



Reaching goals of W7-X within FP9

