

PWIE Meeting November 2025

TOMAS: Status and future plans

Kristel Crombé and the TOMAS team LPP-ERM/KMS

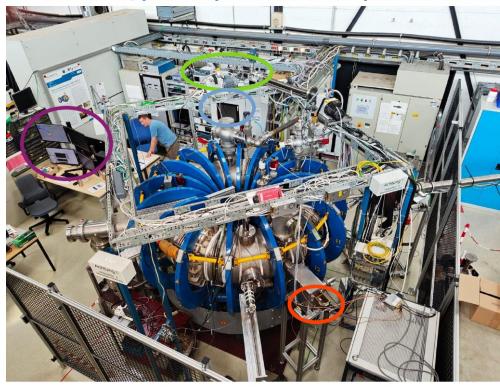






TOMAS device

The TOroidal MAgnetized System (TOMAS) plasma facility aims at complementary research on wall conditioning methods, plasma production and plasma-surface interaction studies.



| Parameter* | Value |
|---------------|--------------|
| Volume | 1.1 m³ |
| Major Radius | 0.78 m |
| Field on axis | 125 mT |
| Minor Radius | 0.26 m |
| RF frequency | 10–50 MHz |
| RF power | up to 6.0 kW |
| EC frequency | 2.45 GHz |
| EC power | 0.6–6.0 kW |
| GD power | 9 kW |
| | |

Diagnostics:

- Horizontal and vertical Langmuir probes
- Microwave Interferometer
- Quadrupole Mass Spectrometer
- Time-of-Flight Neutral Particle Analyzer
- Residual Field Energy Analyzer
- NIR/VIS/UV spectrometer
- Video cameras (incl. fast camera)

Auxiliary systems:

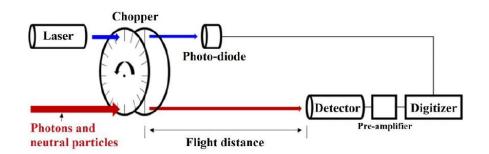
- Control system (Siemens SIMATIC S7-1500 based)
- Data Acquisition system (NI PXIe-1071 based)
- Experiment control system (2 x Arduino Mega & 2 x Tektronix AFG 3252)

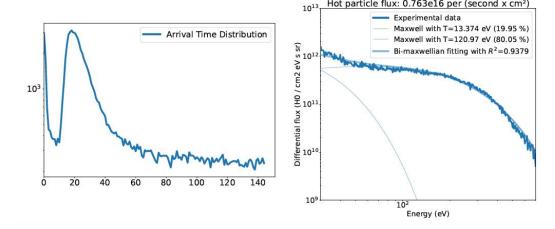
^{*} A. Goriaev et al., Rev. Sci. Instrum. 92 (2021)



Focus on PWIE most relevant diagnostics: ToF NPA and RFEA

Time of Flight Neutral Particle Analyzer → time and energy resolved neutral particle distribution

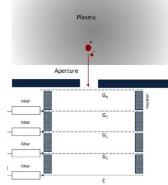


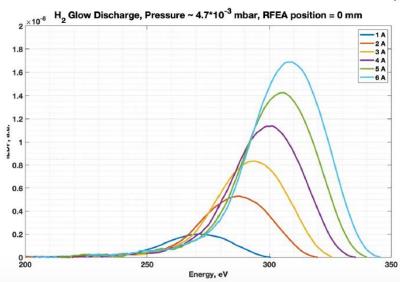


* D. López-Rodríguez et al., Review Scientific Instruments 95, 083542 (2024)

Retarding Field Energy Analyzer
ion fluxes and ion energy
distribution

Ion energy range is 0 – 2000 eV and ion flux range is 0.01 – 50 A/m²

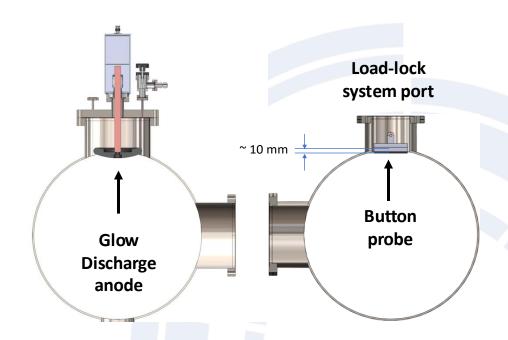


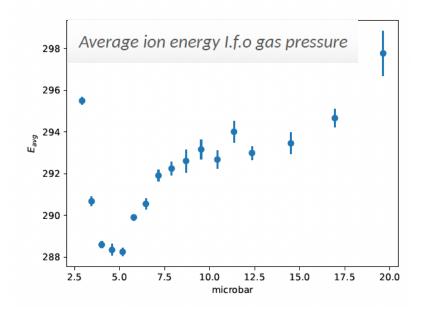


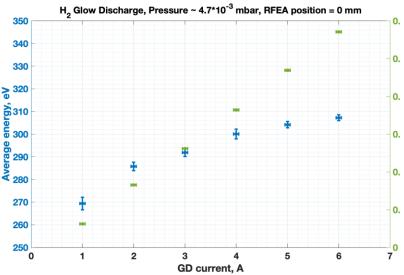


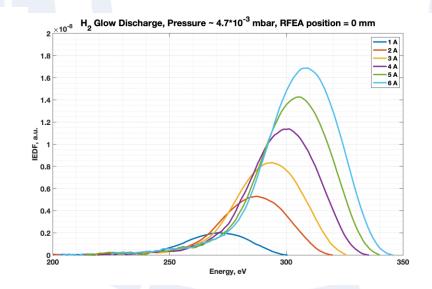
Glow discharge characterization

- Use of RFEA with the new button probe
- Local measurements of average energy and ion flux
- Pressure, current and probe position scans
- Calculation of corresponding boron erosion rates
- Selection of **optimal parameters** for exposure experiment
- ITER relevant pressure, current and sample position





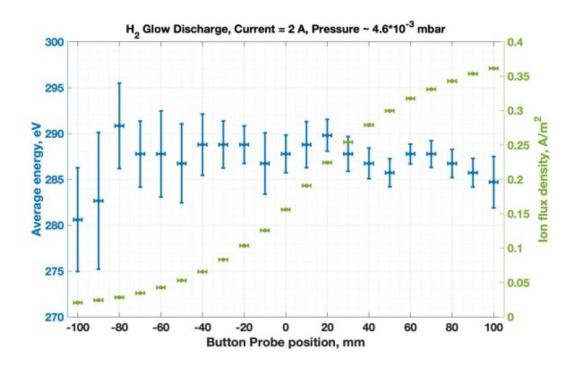


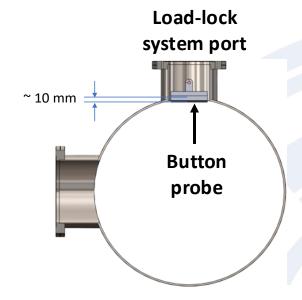


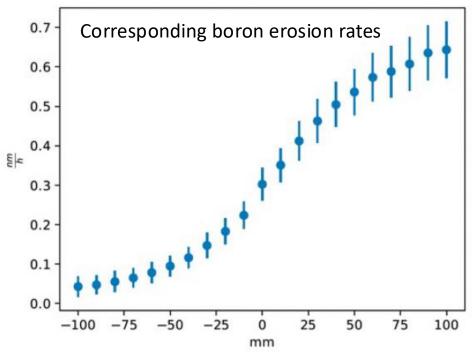


Glow discharge characterization

Average ion energy and ion flux density dependence on holder position



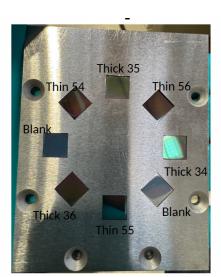


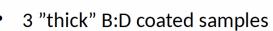




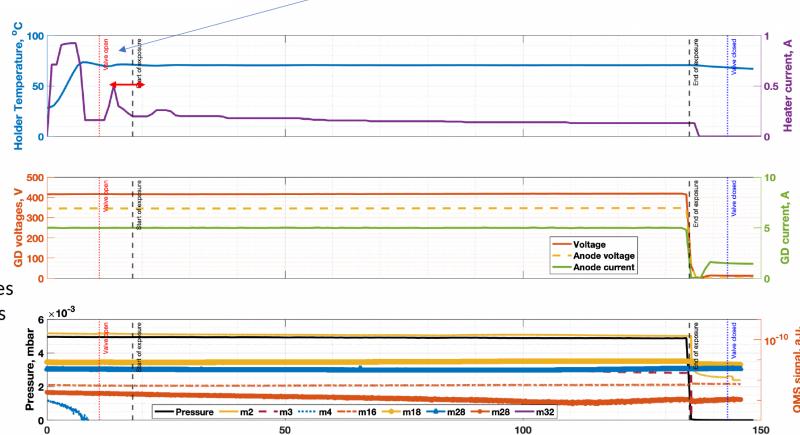
GDC exposure at 70°C

Sample holder movement from idle position to Vessel top

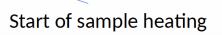




- 3 "thin" B:D coated samples
- 2 dummy clean W samples



Relative time, min



Heating duration at 70 °C: 135 minutes

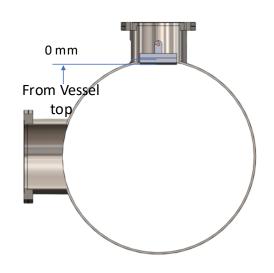
Temperature ramp up: 6 minutes

Pure GD exposure duration: 117 minutes



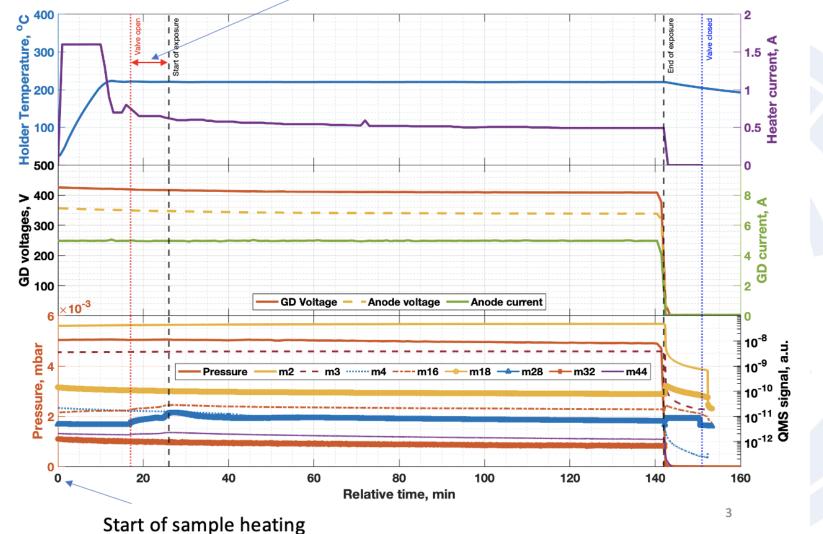
GDC exposure at 220°C

Sample holder movement from idle position to Vessel top



- 4 "thin" B:D coated samples
- 4 dummy clean W samples

Heating duration at 220 °C -> 131 minutes Temperature ramp up -> 12 minutes Pure GD exposure duration -> 117 minutes



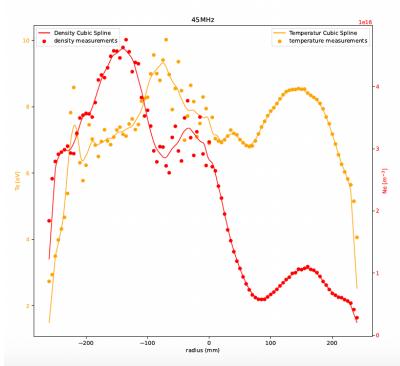


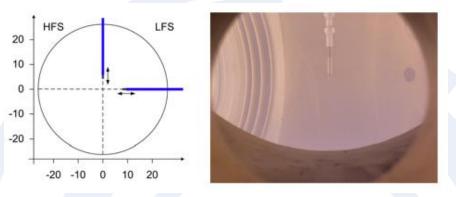
Ion cyclotron wall conditioning characterization

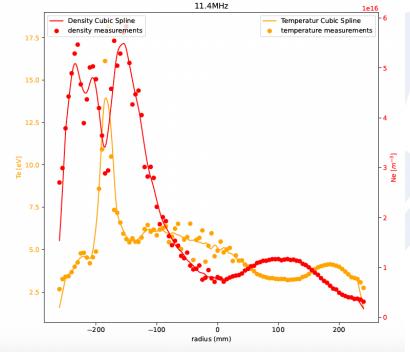
- Pressure, RF frequency and magnetic field scans
- Measurements of average energy and flux of CX neutrals
- Calculation of corresponding boron erosion rates
- Selection of optimal exposure experiment parameters
 - Exposure duration, ICWC duty cycle
 - > ITER relevant pressure, RF frequency and magnetic field scans



High Power High Pressure







Low Power

Low Pressure

25

High Power

Low Pressure



ICWC exposure at 70°C

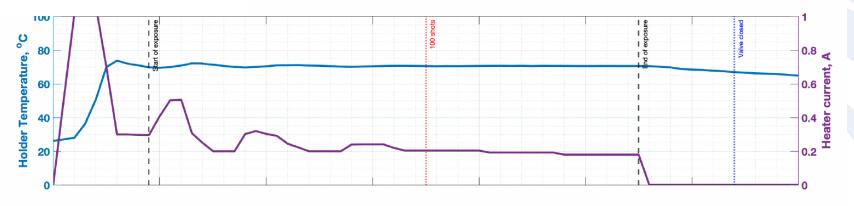
Parameters:

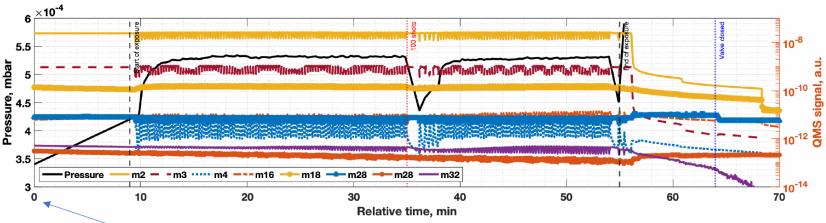
RF frequency -> 45 MHz, Magnetic field -> 1600 A, IC pulse length -> 4 s, dwell time -> 10.9 s





- 2 "thin" B:D coated samples
- 4 dummy clean W samples





Start of sample heating

Heating duration at 70 °C: 50 minutes

Temperature ramp up: 6 minutes

Pure IC exposure duration:11.3 minutes

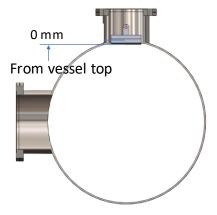


ICWC exposure at 70°C

Repeated for longer time

Parameters:

RF frequency -> 45 MHz, Magnetic field -> 1600 A, IC pulse length -> 4 s, dwell time -> 10.9 s, 510 shots





5 dummy clean W samples

Heating duration at 70 °C

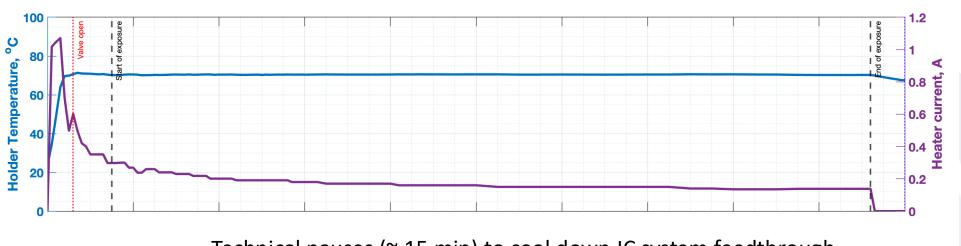
-> 186 minutes

Temperature ramp-up

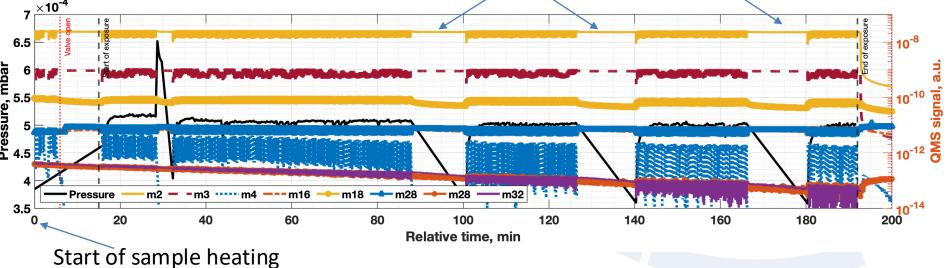
-> 6 minutes

Pure ICWC exposure

-> 34 minutes

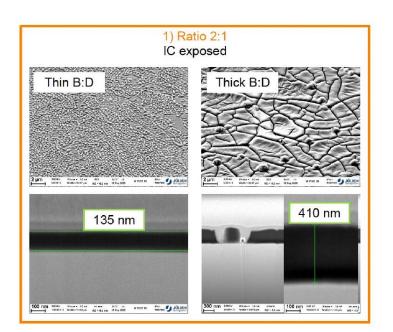


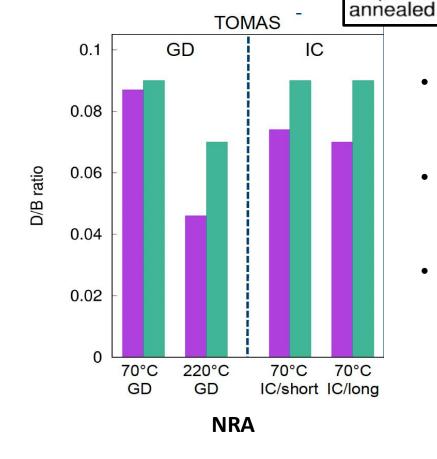






B:D ratios and isotope exchange





 H2 GDC combined with baking at 220°C reduces deuterium content by 43%

exposed

- Only annealing at 220°C reduces deuterium content by 30%
- IC long: D reduction is not in correspondence with 3 times longer exposure

FIB/SEM measurements

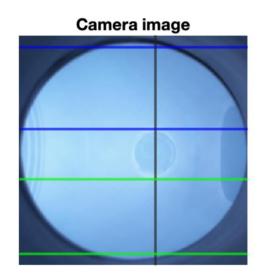
- Clear indication of deuterium reduction in the exposed B:D layers -> further investigation
- Very low erosion of B:D Layer in TOMAS (GDC and IC) -> within FIB/SEM measurement error
- Thick IC/annealed: cracks and delamination due to heat/exposure
- Strong indications of water uptake of B:D layers

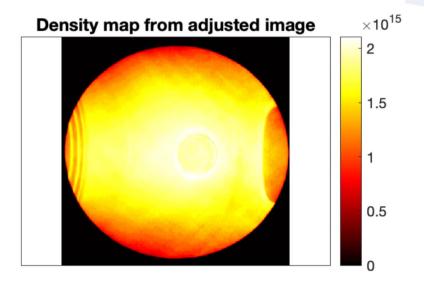


Glow discharge characterization

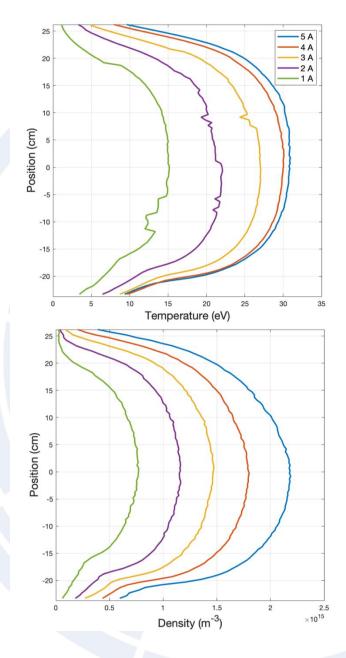
Measurement of electron density and temperature profiles

- Combining camera images and probe measurements for estimating
 2D density map
- Ongoing master thesis project by M. Hillen (TU Eindhoven): autumn 2025 – beginning 2026





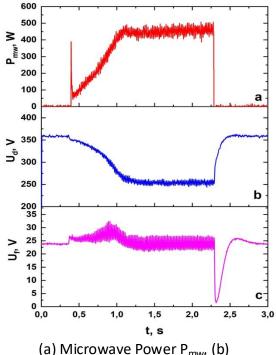
N. Desmet, "Determination of the Vertical Density and Temperature Profiles in an ECRF Plasma in the TOMAS Device", Master thesis, LPP-ERM/KMS (2023)



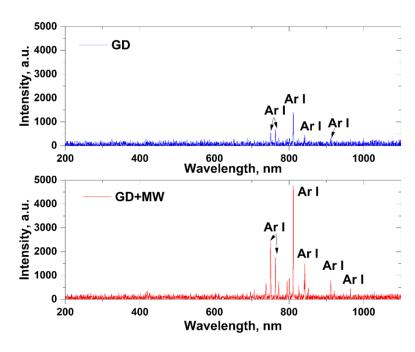


Contribution of KIPT (1)

Experiments were conducted on **combined glow and microwave discharge**, in an atmosphere of argon and helium. Data processing ongoing.



(a) Microwave Power P_{mw} , (b) discharge voltage U_d , (c) floating potential U_f . Ar gas, pressure 7 Pa, probe at r = 2 cm.



Optical emission spectra (GD 350 V, $P_{MW} = 500 W$)

Observations:

- Injection of microwave power allows to decrease the voltage GD (decrease ≈ 100 V at microwave power 0.5 kW). Stronger decreased possible with higher microwave power.
- Intensity of spectral lines of excited argon atoms in combined glow + microwave discharges also increases compared to GD only.
- Energy of ions decreases as the voltage decreases

Results of previous experiments:

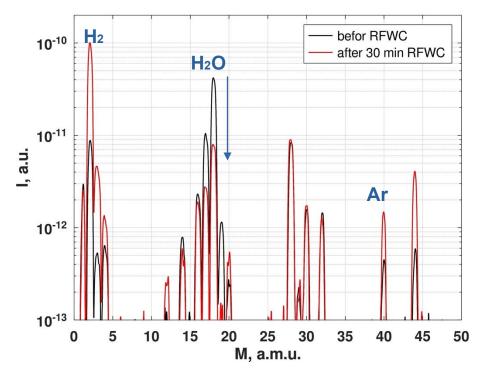
Y. P. Martseniuk et al., Combined glow + microwave discharges in the TOMAS plasma facility, Journal of Fusion Energy, (2025) 44:49



Contribution of KIPT (2)

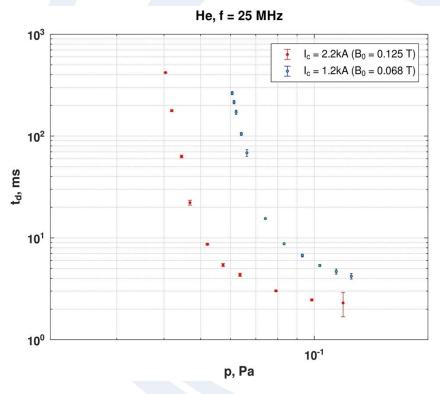
Radio Frequency RF Wall Conditioning (RFWC) in argon at frequencies significantly higher than resonant ion cyclotron frequencies $\omega_{\text{RF}} \gg \omega_{\text{ci}}$.

Decrease in partial pressure of water by RFWC.



Partial pressures of residual gas components. RF f = 25 MHz, power 2 kW. B(0) = 0.1 T

RF plasma start-up at various frequencies, pressures, and magnetic fields in argon and helium. **Decrease in breakdown time** with an **increase in pressure and magnetic field**.



Dependence of RF breakdown time on pressure.



Use of existing diagnostics

• Employ Langmuir probes, ToF NPA, RFEA, OES, visible cameras, and RGA to study wall-conditioning plasmas and support sample-exposure experiments, delivering data on **erosion**, **deposition**, and **impurities**.

Extension of Diagnostic Capabilities

- PSI-2 Integration: Connect TOMAS to the PSI-2 **LID-QMS** for sensitive, species-selective analysis of desorbed and eroded particles.
- Laser Compatibility: Upgrade TOMAS for laser operation (optical closure, safety) and install a new dedicated laser system.
- **Spectrometers**: Adapt and reinstall JET overview and high-resolution spectrometers with new optical interfaces tailored to TOMAS.



Future physics studies

- **PSI on boron layers**: Deuterium-doped boron layers, including impurity-containing mixtures, will be exposed to **ICWC** and **GD** plasmas in TOMAS. A **heated sample holder** will recreate realistic wall conditions to continue the study of fuel retention and impurity behaviour.
- ITER-relevant GD upgrades: A second GD anode will be added to replicate ITER's boronization geometry. Gas injection will be reconfigured to test H, He, and H/He mixtures for optimized cleaning and conditioning, supported by modelling from INP Greifswald.
- In-situ laser diagnostics: New laser-based analysis will provide direct monitoring of surface modification and fuel retention during and after plasma exposure.
- Sample testing for W7-X: Boron-coated CFC samples for W7-X will be assessed for coating stability and erosion. Pre-characterized graphite divertor samples from W7-X (FZJ) will be exposed in TOMAS and subsequently re-analyzed.
- **ECWC plasma studies**: Further experiments and modelling, with emphasis on parametric decay, mode conversion, and tangential injection effects.



Recent publications and achievements

- A. Adriaens et al., "An automatic matching system for the ICRF antenna at TOMAS: Development and experimental proof", Fusion Engineering and Design 212, 114840 (2025)
- Yu. Kovtun et al., "Combined electron-cyclotron resonance and radio-frequency discharges in the TOMAS facility", Physics of Plasmas 32, 032512 (2025)
- Y. P. Martseniuk et al., "Combined Glow + Microwave Discharges in the TOMAS Plasma Facility", Journal of Fusion Energy (2025) 44:49
- J. Buermans et al., "Mode Conversion and Parametric Decay Instabilities in Electron Cyclotron Resonance Heated Helium and Hydrogen Plasmas in TOMAS ", EPS Poster (2025)
- A. Adriaens et al., "Wall conditioning on TOMAS with relevance to ITER and W7-X", ICRFM Poster (2025)
- A. Goriaev, A. Adriaens, et al., "Laboratory Assessment of ITER-Relevant Boron Layer Fuel Removal using Glow Discharge Cleaning and Ion Cyclotron Wall Conditioning in the TOMAS Device", PMIF, Oral presentation (2025)
- B. Bozonnet, Sorbonne University, France, internship on simulation of neutral particle fluxes in TOMAS NPA diagnostic (2025)
- M. Hillen, TU/Eindhoven, master thesis on camera system as a new diagnostic tool for plasma density measurements, ongoing (autumn2025 – beginning2026)





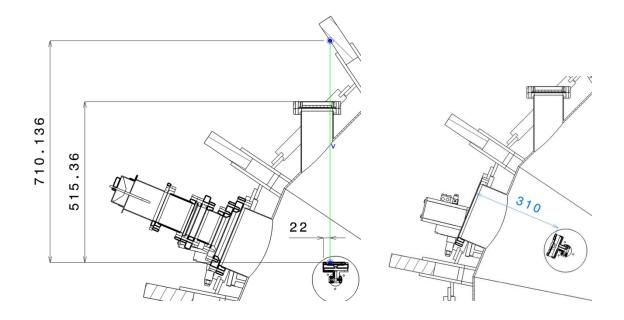


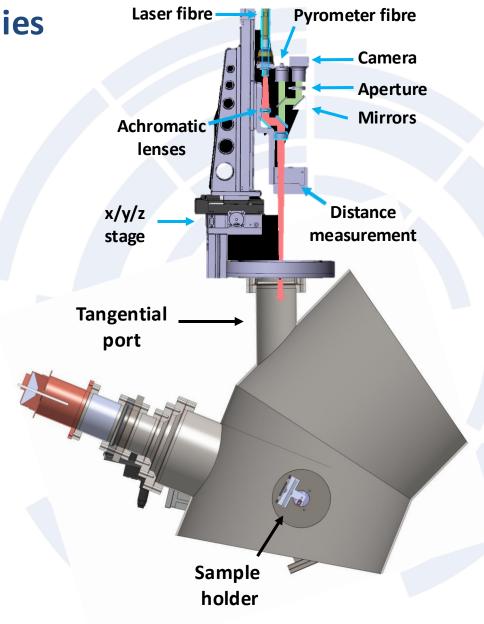




Implementation of in-situ boronization studies

- Preparation for the installation of laser-induced desorption quadrupole mass spectrometry (LID-QMS)
 - Adaptation of laser-detector geometry
 - Design and assembly of support structures for fiber optics
 - Selection of optical elements





M. Zlobinski et al., Nuclear Fusion 64, 086031 (2024)



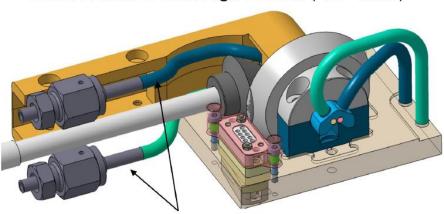
Upgrade of sample load lock

The new design of the material sample manipulator

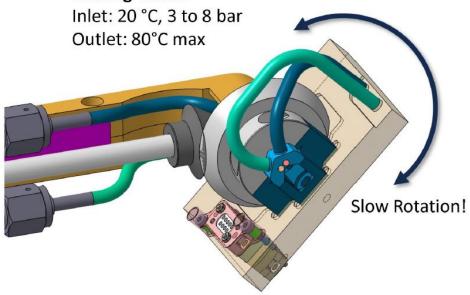
- Actively cooled sample holder
- Fully automated 3d manipulator movement
- Fast and easy exchange of the manipulator holder
- Extended manipulation range
- Upgraded differential pumping



Outer Pressure: Ultra-High Vacuum (<10⁻⁷mbar)



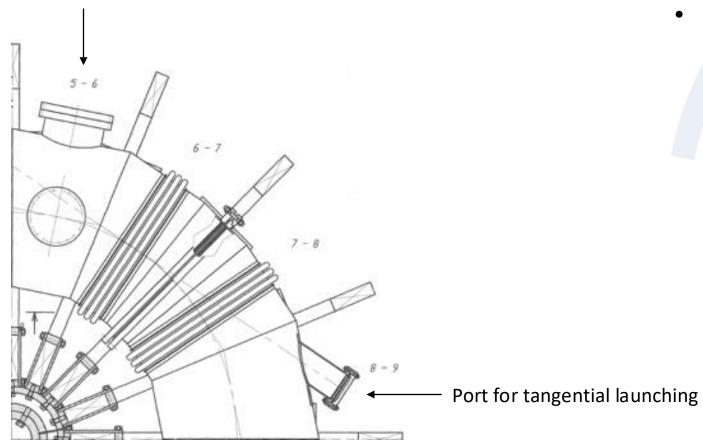
Cooling water:





Upgrades to the heating systems

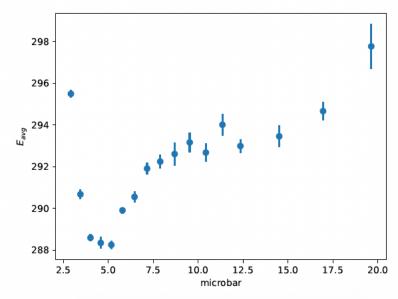
Current ECRH launching port



- Extension of the ECRH system
 - Addition of a 2 kW gyrotron to increase the total power
 - Use of the tangential port
 - Installation of waveguide tuners, mode convertors, polarizers, vacuum windows, cooling elements
 - Integration of the new gyrotron to the control system

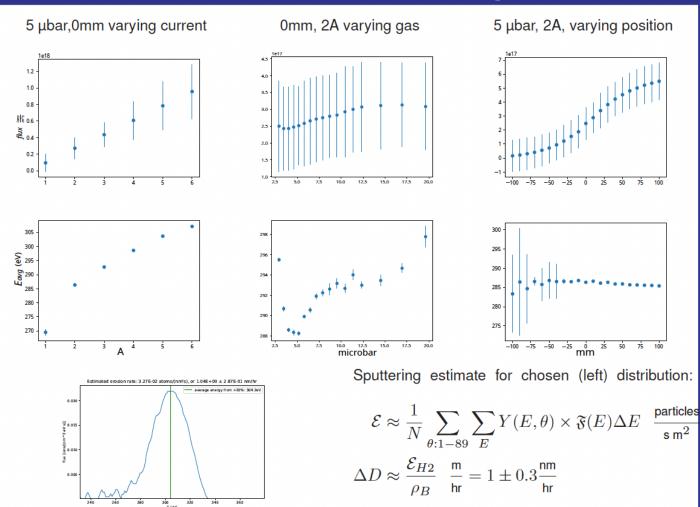


Erosion rate calculations



Average ion energy I.f.o gas pressure

H2 GDC Plasma Characterization using a RFEA

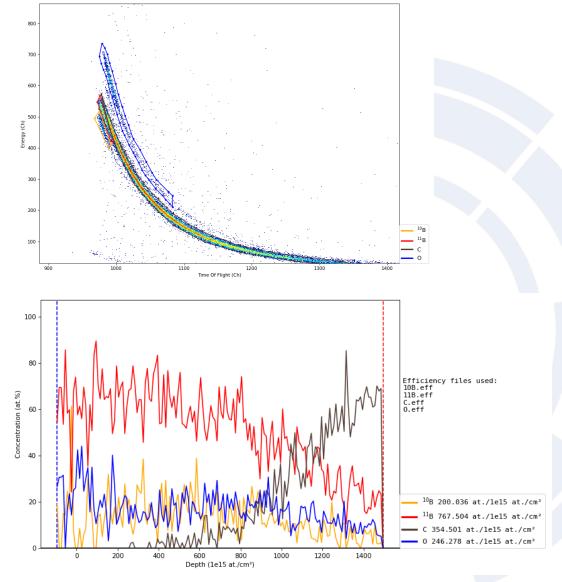


With $Y(\theta,E)$ the angle of incidence and energy-dependent yield (computed using the code rustBCA), $\mathfrak{F}(E)$ the flux per energy bin and N the amount of angles considered (assumed homogeneous). As the layer is 100nm thick, 2 hours of exposure should leave 98% intact.



Studies of boron layer erosion

- Pre-characterization and post-mortem analysis
 - Boron coating thickness by Focused Ion Beam Scanning Electron Microscope (FIB/SEM) and ellipsometry
 - Boron coating roughness by profilometry
 - Boron coating elemental composition by ion beam analysis
 - ➤ Elastic Recoil Detection Analysis (ERDA)
 - Nuclear Reaction Analysis (NRA)
 - Analysis of composition by Thermal Desorption Spectroscopy (TDS)
 - Monitoring of impurity release during exposure by Quadrupole Mass Spectrometry (QMS) and optical spectrometry



Results obtained by ERDA