

# TSVV-12: stellarator optimization

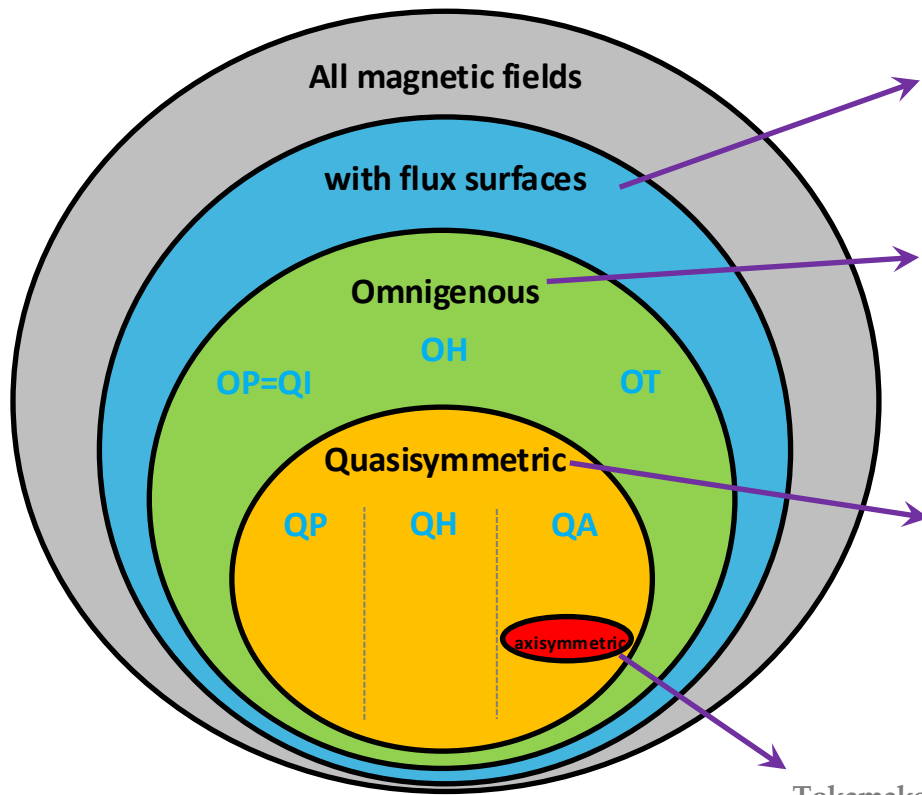
Joaquim Loizu *on behalf of the TSVV-12 team*

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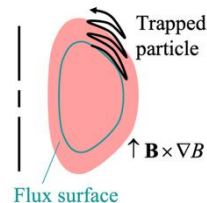
<sup>1</sup> IPP, <sup>2</sup> Aalto University, <sup>3</sup> TU Graz, <sup>4</sup> EPFL, <sup>5</sup> PPPL, <sup>6</sup> CIEMAT



# Where to optimize in the space of all magnetic fields?

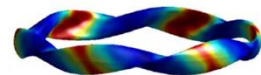


does not guarantee confinement of trapped particles

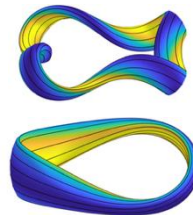


net radial drift = 0

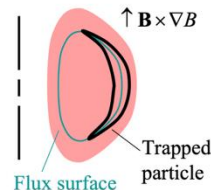
- $|\mathbf{B}|$  contours need to close OP/OH/OT
- no easy recipe
- W7-X is approximately OP=QI



- $|\mathbf{B}| = |\mathbf{B}|(\psi, M\theta - N\phi)$  in Boozer coordinates
- more easy recipe
- QA neoclassical physics = tokamak



Tokamaks





# Traditional stellarator optimization

1<sup>st</sup> stage optimization:  
Plasma boundary



**Slow** routines  
evaluate more  
properties of  
equilibrium /coils



2<sup>nd</sup> stage optimization:  
Coil Geometry

Optimizer  
varies the  
*plasma boundary*  
and evaluates a  
target function

**VMEC**  
requires the  
*Fourier harmonics*  
of the plasma  
boundary as input

**Fast** routines  
evaluating  
properties of  
equilibrium

Optimizer  
varies the  
*coil geometry*  
and *currents*  
and evaluates a  
target function

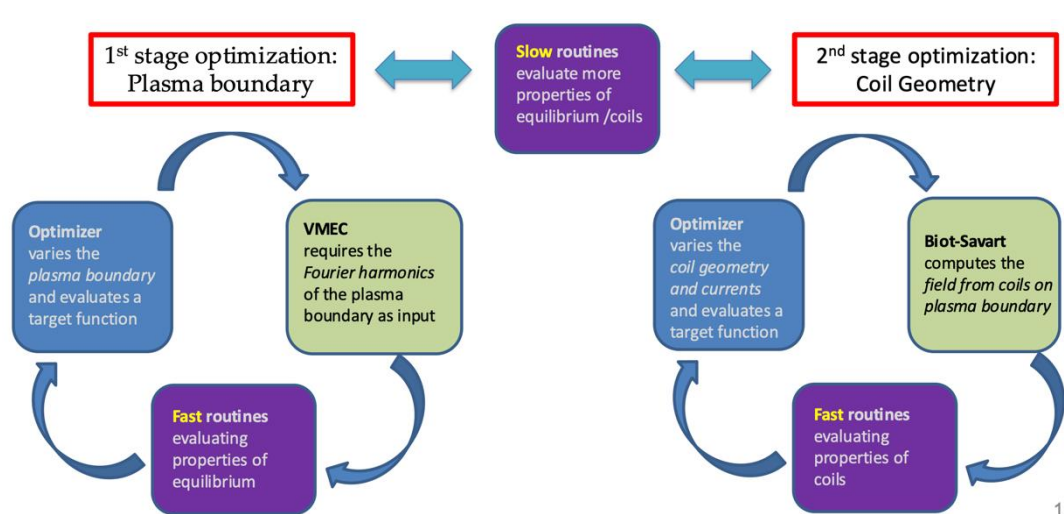
**Biot-Savart**  
computes the  
*field from coils on*  
*plasma boundary*

**Fast** routines  
evaluating  
properties of  
coils



# What is different today that allows much better designs?

- Much faster computing (can explore more space)
- Devised efficient recipes for finding QI and QS (better target functions)
- Developed faster codes (can include more evaluations inside optimization loop)
- Derived new reduced models (more physics in optimization loop)
- Combined plasma-coil optimization or “single-stage” (potentially accessing much better optimum)

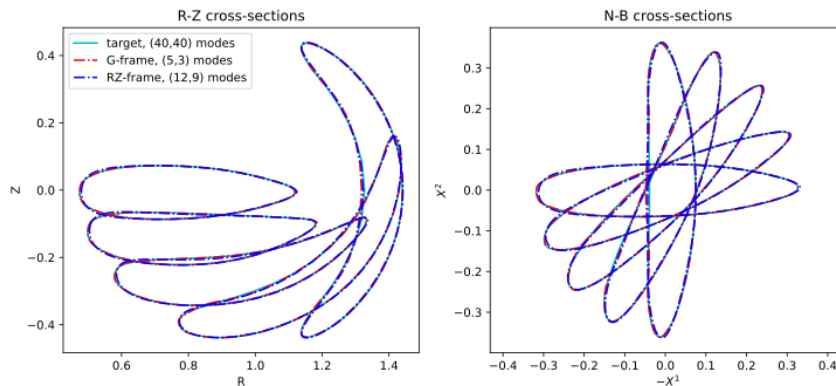
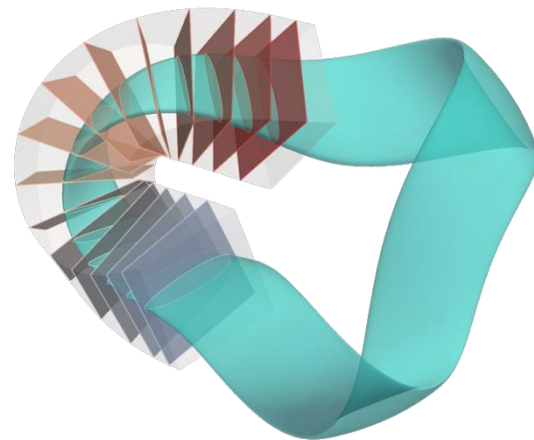


[Henneberg et al, JPP 2021]



# Developed robust ideal MHD equilibrium code (GVEC)

- **GVEC** improves upon VMEC on many aspects (numerics).
- General coordinate frame “**G-frame**” implemented in GVEC.
- G-frame aligns planar cross-sections (not R-Z) with plasma shape.
- Reduces degree-of-freedom to represent equilibrium.
- **Enables exploration** of strongly shaped configurations!

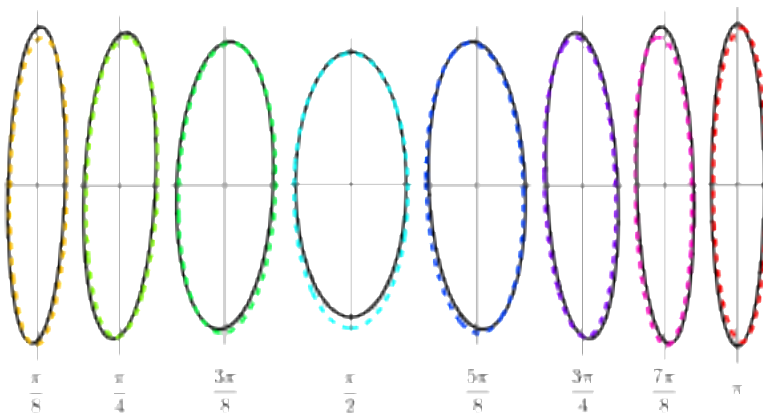
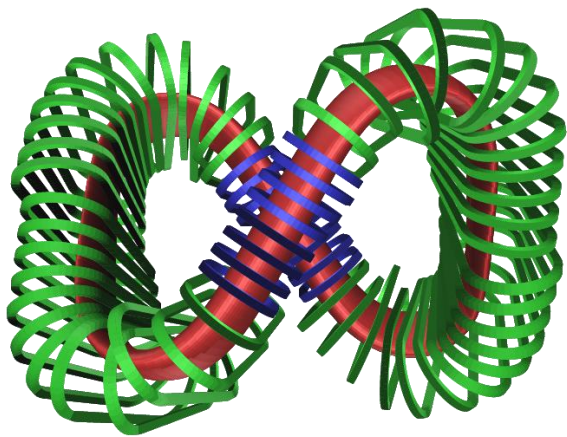
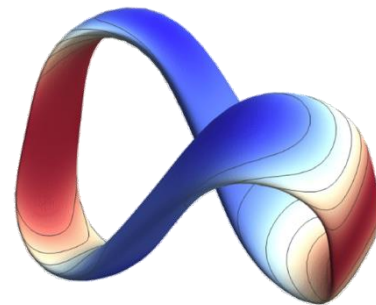


[Hindenlang et al, PPCF 2025]



# First ever QI stellarator design with planar coils

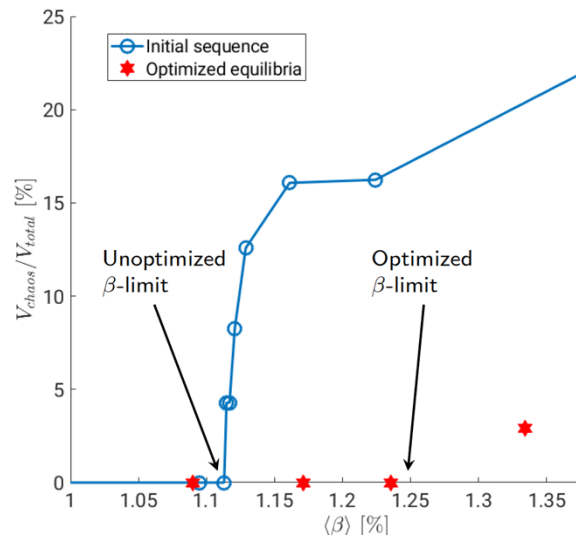
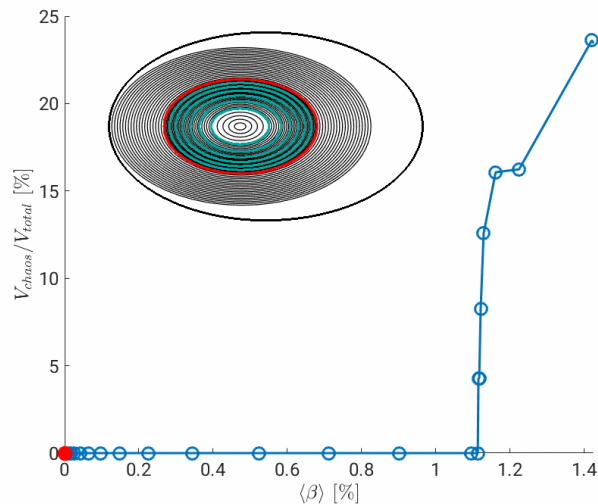
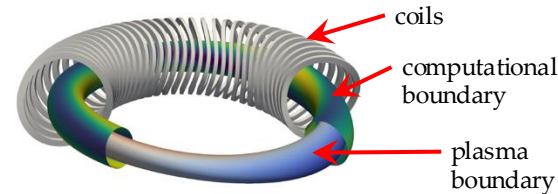
- GVEC integrated into optimization code SIMSOPT.
- QI-optimized stellarator with **planar coils**!
- It's a **figure-8** with nearly-elliptical cross-sections and stable magnetic well.





# SPEC used to quantify & optimize equilibrium $\beta$ -limits

- **SPEC** allows fast calculation of free-boundary equilibria with islands & chaos, including effect of bootstrap current. [Baillod et al, JPP 2021]
- Quantified  $\beta$  above which volume of chaos  $V_{\text{chaos}} > 0$ . [Baillod et al, JPP 2023]
- SPEC integrated with SIMSOPT, allowing topology optimization. [Baillod et al, PoP 2022]



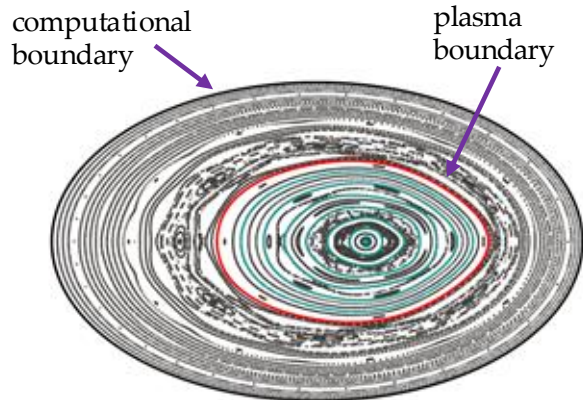




# Efficient single-stage optimization of islands at finite- $\beta$

**Objective:** minimize  $R^2_{\text{Greene}}$  for selected resonances

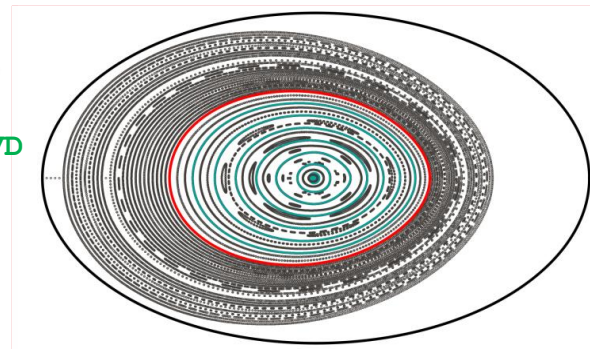
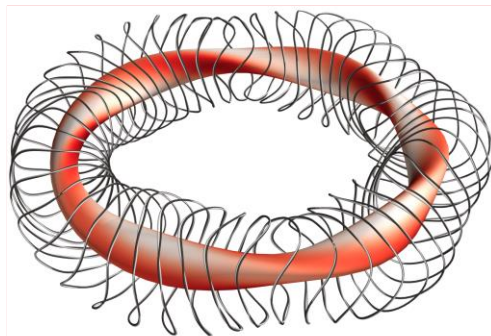
**Degrees of freedom:** coils geometry  $\mathbf{x}_{\text{mn}}$  in Fourier



**initial**  
( $\beta \sim 1.5\%$ )

coupling SIMSOPT + SPEC

80-dim optimization space with SVD



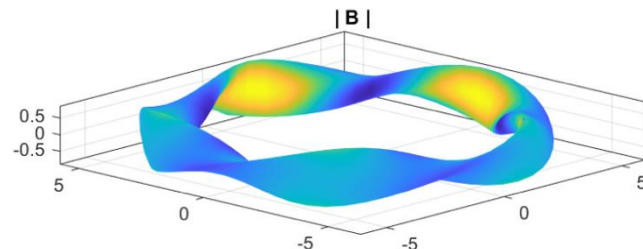
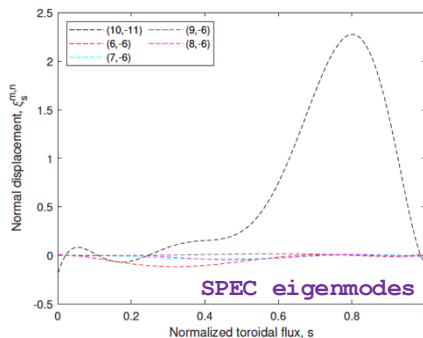
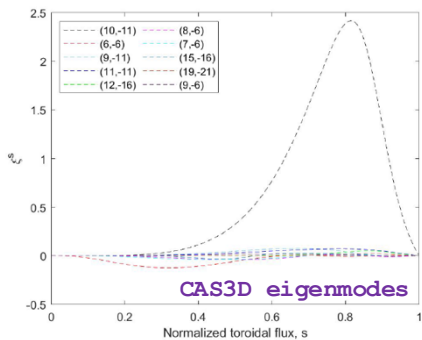
**optimized**  
( $\beta \sim 1.5\%$ )



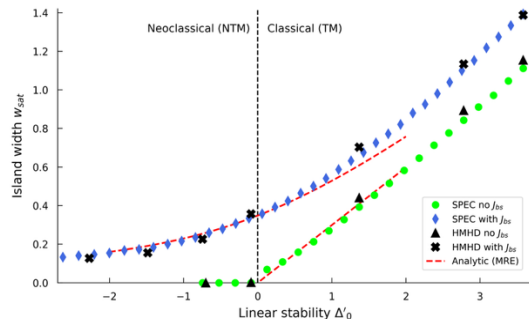
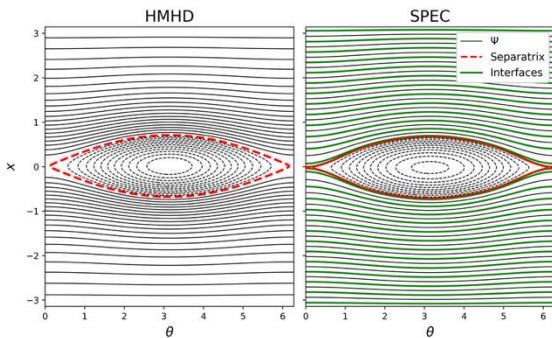


# MHD linear/nonlinear stability with an equilibrium code

- Retrieved ideal MHD stability in a stellarator with SPEC. [Kumar et al PPCF 2022]



- Reproduced nonlinear saturation of tearing modes with SPEC. [Loizu et al JPP 2023; Balkovic PPCF 2024]



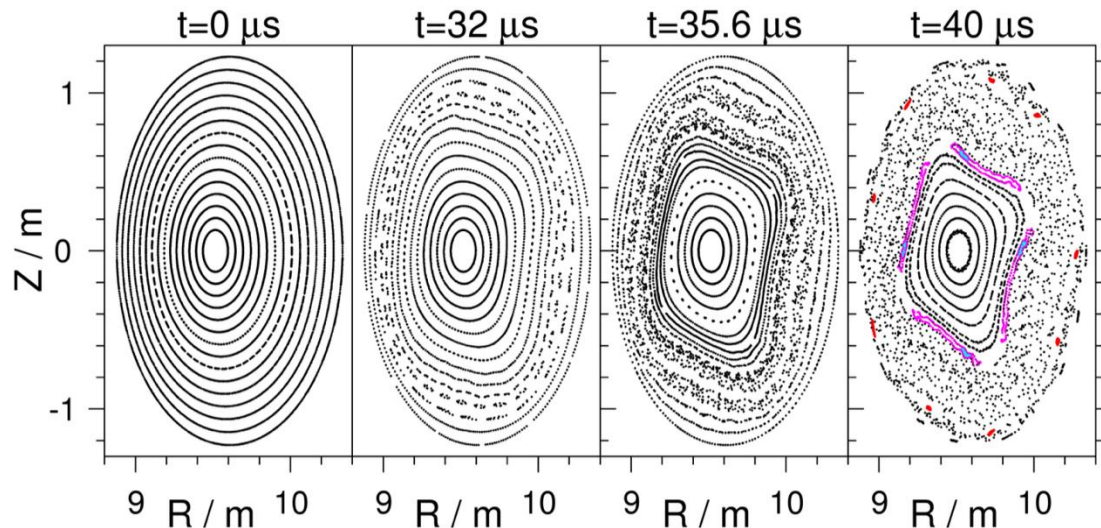
**Resistive MHD simulation:**  
400 cpu-hours

**Direct solve with SPEC:**  
0.1 cpu-hours



# MHD linear/nonlinear stability with a gyrokinetic code

- Global GK simulations of MHD-unstable low-shear stellarators with **EUTERPE**.



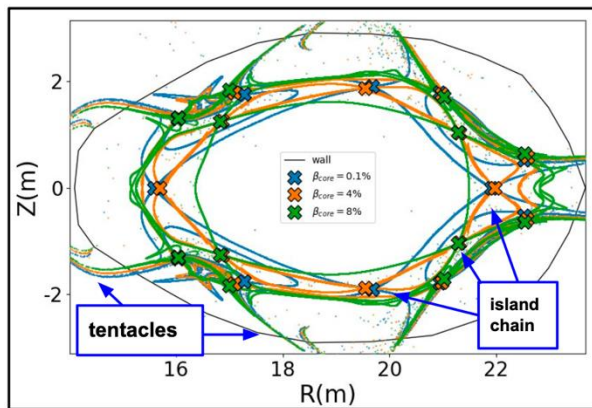
- Linear phase agrees with ideal MHD stability predictions (CAS3D)
- Nonlinearly, low- $m$  modes can drive islands and ergodization, potentially affecting confinement

[Nührenberg et al JPP 2025]



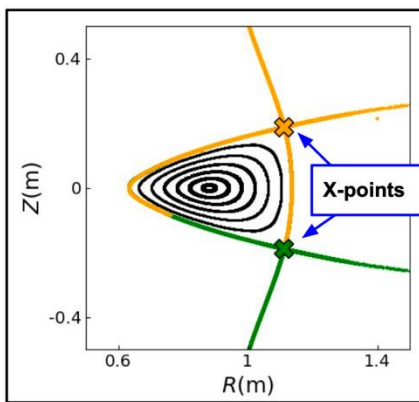
# Topological methods used to monitor & control the edge

- Diversion in stellarators largely governed the existence & properties of fixed points (X-points/O-points).
- Topological methods used to understand and optimize fixed points to improve stellarator divertors.



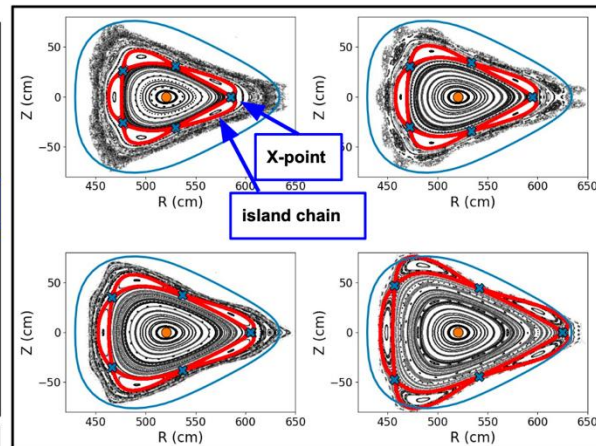
## Edge topology of SQUID reactor concept

Island chain present which flips phase as  $\beta$  increases  
Additional X-points divert plasma in apparently resilient "tentacles"



## Discovery of stellarator with "tokamak-like" X-points

Starting point for design of STAR\_Lite at Hampton University, Virginia!



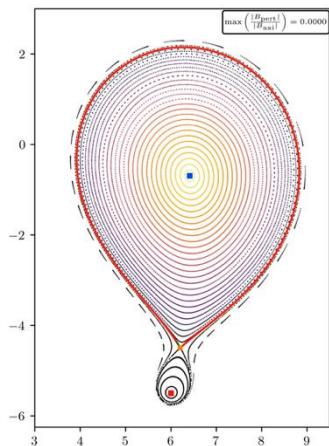
## Topological optimisation in W7-X

Using optimisation schemes to "automatically" move the island chain by targeting X-point location

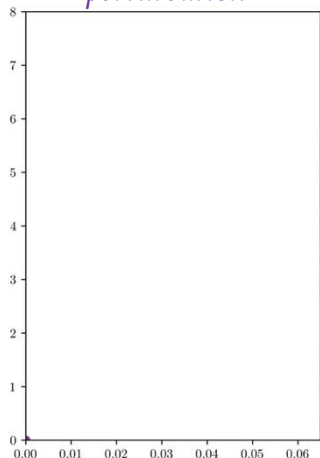


# We can now compute manifolds & turnstile lobe areas

*manifolds in a  
perturbed  
tokamak*

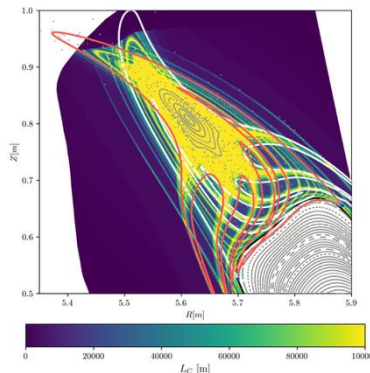
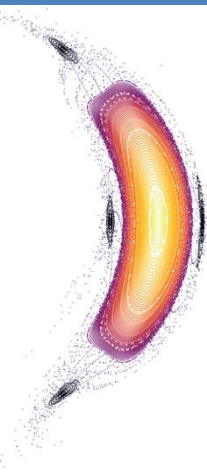
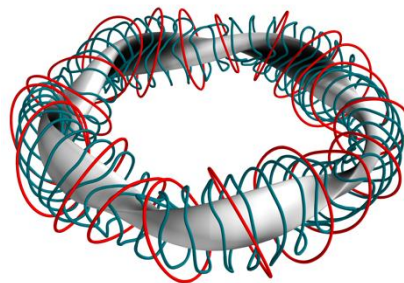


*turnstile area  
versus  
perturbation*



As the perturbation strength increases, islands appear and merge, and Poincaré trajectories jump, but the manifolds and turnstile areas vary smoothly.

All tools are being integrated in simsopt!



In W7-X configurations where the divertor island region is chaotic, intersecting turnstile lobes coincide with low connection length regions.

[Smiet et al, Chaos 2025]



# Developed EMC3-Lite code for fast heat load evaluation

Heat transport equation solve by EMC3-Lite:

$$\nabla \cdot (-\kappa_e \nabla_{\parallel} T - \chi n \nabla_{\perp} T) = 0$$

Bohm condition at target:

$$q_P = -\kappa_e \nabla_{\parallel} T = n C_s \gamma T_t$$

Numerics:

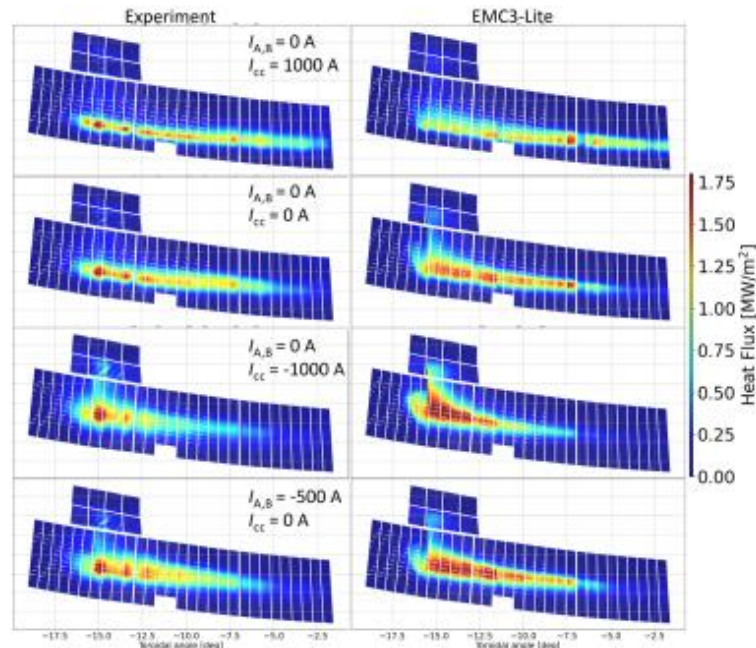
*Monte-Carlo integrated with  
Reversible Field-Line Mapping (RFLM)*

Major features:

- *$10^3$  times faster than conventional diffusive-field-line-tracing*
- *heat load distribution including the target shadow region*

[Feng et al PPCF 2022]

## Validation at W7-X



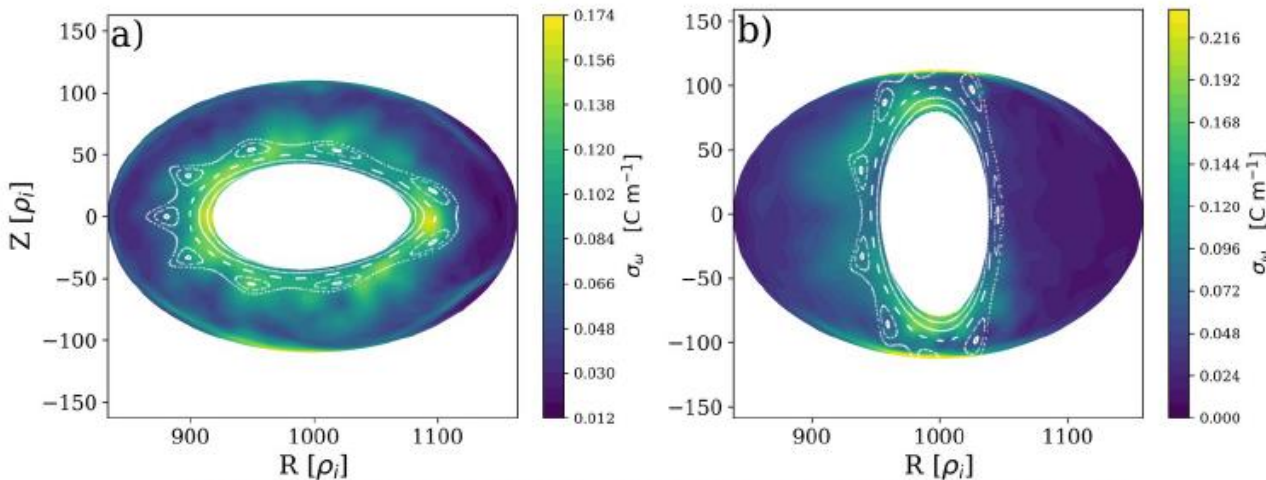
[Gao et al NF 2023]





# Extended BOUT++ for stellarator edge turbulence

- Fluid turbulence simulations have been performed with **BOUT++** in the edge of an analytic stellarator configuration with an island divertor. [Shanahan et al JPP 2024]
- Synergy with TSVV-3 work, with GBS simulations of the same configuration. [Coelho et al NF 2022]

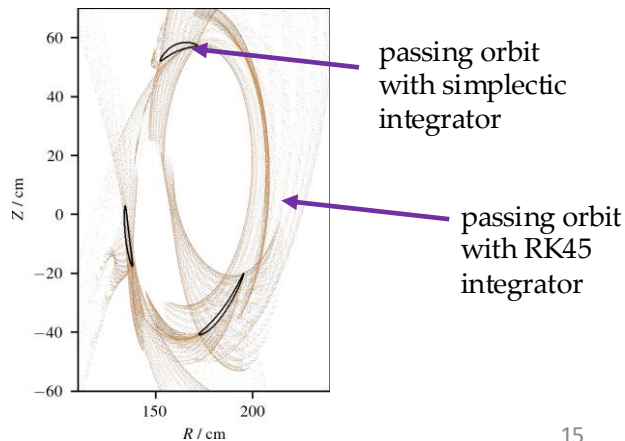
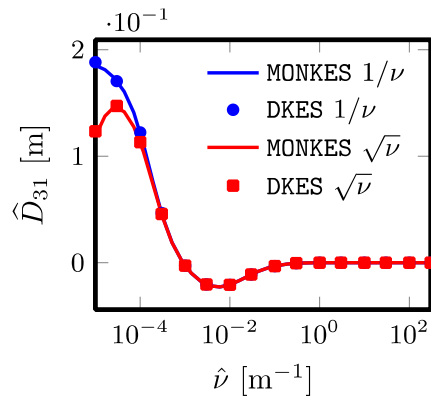
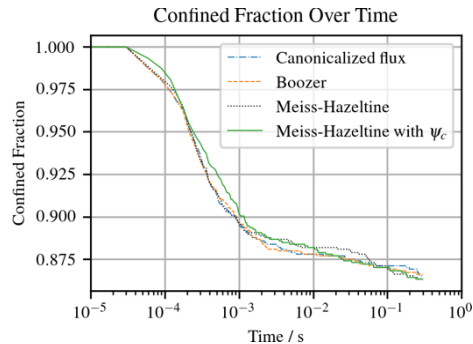


- Opens the way to V&V studies for stellarator edge turbulence simulations. [Coelho et al NF 2023]



# Fast & accurate neoclassical and orbit-following codes

- **MONKES**: accurate neoclassical code [Escoto et al NF 2024]
  - very fast (< minute / point, using 1 core )
  - evaluation of monoenergetic transport coefficients
  - bootstrap current calculation can be put into optimization loop
- **SIMPLE**: fast  $\alpha$ -particle orbit follower with symplectic integrators
  - long-term stability at large time steps [Albert et al PPCF 2025]
  - applied to calculate confined fraction of  $\alpha$ 's in a SQuID
  - applied to study alpha particle heat loads on a reactor wall



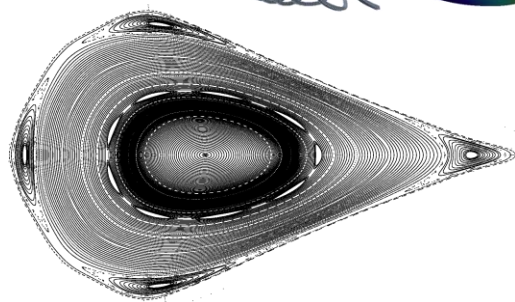
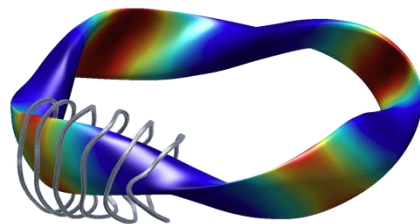




# CIEMAT-QI family of reactor-relevant configurations

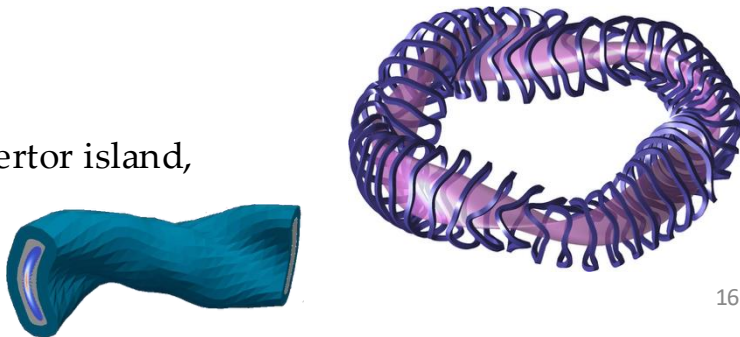
**CIEMAT-QI4, first QI configuration** with: [Sanchez et al NF 2023]

- $\iota$  profile avoids low order rationals, compatible with island divertor,
- Ideal MHD stability up to  $\beta = 5\%$ ,
- Low neoclassical transport and bootstrap current,
- Very good fast ion confinement for  $1.5\% < \beta < 4\%$ ,
- Reduced turbulent transport,
- Set of elementary coils preserving properties.



**CIEMAT-QI4X:** [Sanchez et al, submitted NF]

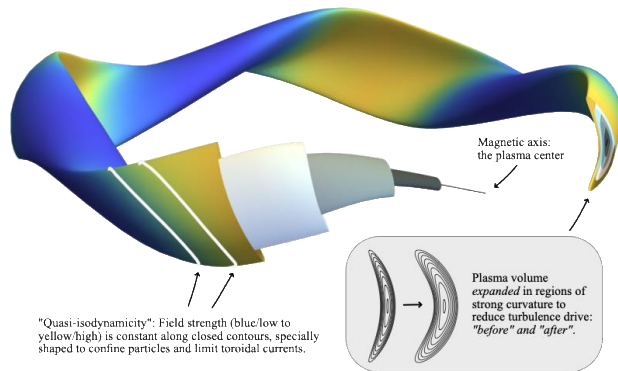
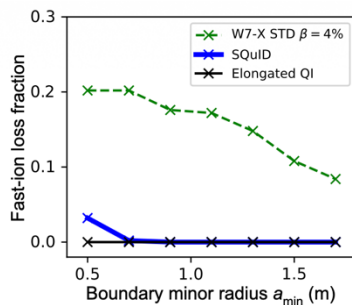
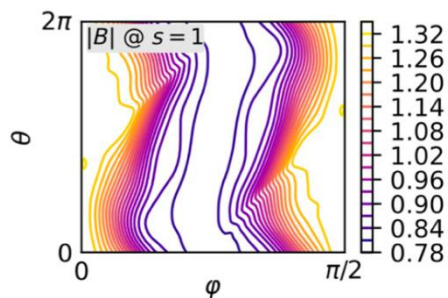
- Improved flux surface quality & robustness of divertor island,
- Simpler coils with larger separation,
- Ongoing study of breeding blanket feasibility.





# SQuID family of QI designs with reduced ITG turbulence

## ➤ SQuIDs: Stable Quasi-Isodynamic Designs [Goodman et al PRX Energy 2024]



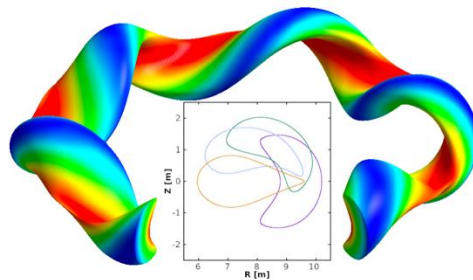
- Very close to perfect QI
- Ideal MHD stability up to  $\beta = 3\%$ ,
- Low bootstrap current ( $\sim 15$  kA)
- Very good fast ion confinement at target  $\beta$
- Reduced turbulent transport
- Filamentary coils with complexity  $\sim$  W7-X



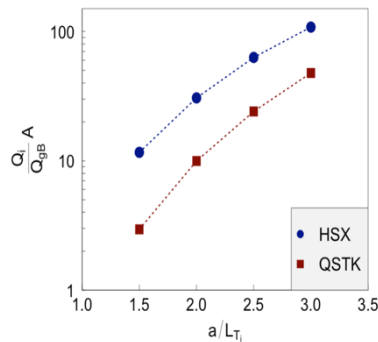
# Turbulence optimization can improve core confinement

## ➤ QSTK: QH with **reduced ITG turbulence**

- Better fast-ion confinement than W7-X
- Lower thermal neoclassical transport than W7-X
- MHD stable

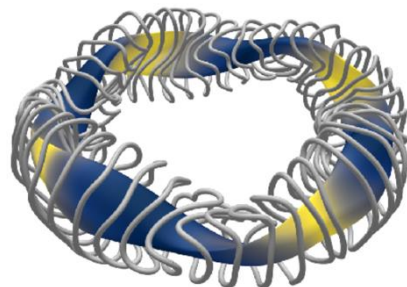


[Roberg-Clark et al PRR 2023]

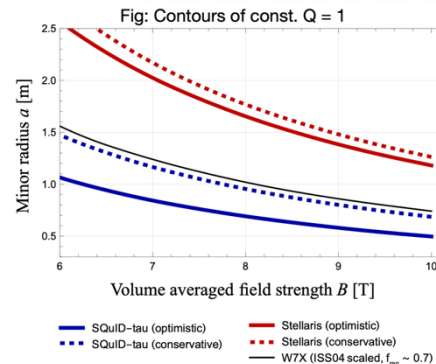


## ➤ SQUID- $\tau$ : stable QI with **enhanced turbulent particle pinch**

- Zero losses (collisionless) of fusion-born alphas.
- Mercier stability to  $\langle \beta \rangle \sim 7\%$
- Coil compatibility
- KBM stability
- Negligible bootstrap current ( $\sim 10$  kA)



[Plunk et al, submitted 2025]





# Can optimize for $E_r > 0$ in the core

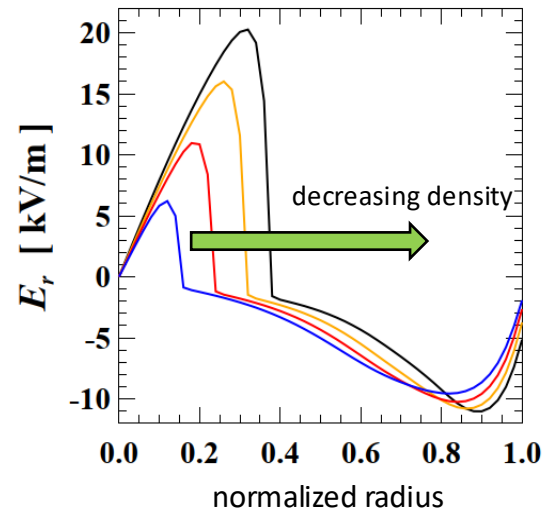
In most stellarators, radial electric field determined by requirement of ambipolar neoclassical transport:

Usually  $E_r < 0$  in stellarators (ion root)

- Causes strong inward neoclassical transport for highly charged impurities.

$E_r > 0$  (electron root) observed in low-density plasmas with  $T_e > T_i$ , but has recently been found to be possible in the core if  $T_e = T_i$ .

- Beneficial for impurity expulsion
- Sharp transition may cause transport barrier.
- Can be achieved through targetted optimisation.



Onset of electron root approximately when

$$D_e^{1/\nu} > D_i^{\sqrt{\nu}} \quad \Rightarrow \quad \frac{T_e}{T_i} \geq \left( \frac{m_i}{m_e} \right)^{1/7} \left( \frac{\epsilon_i \nu_{*i}}{\epsilon_{\text{eff}} \rho_{*i}} \right)^{3/7}$$

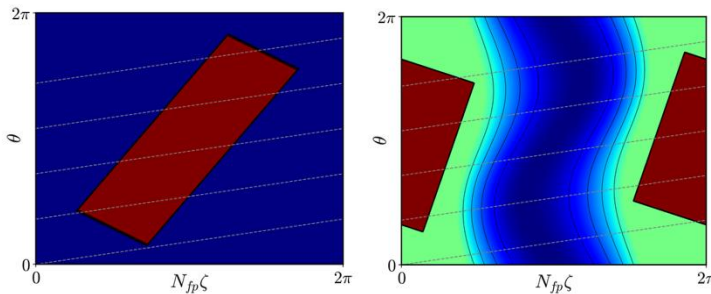
[Beidler et al., Nucl. Fusion 2024], [Helander et al., JPP 2024]



# Discovery of pwO enlarges optimization space

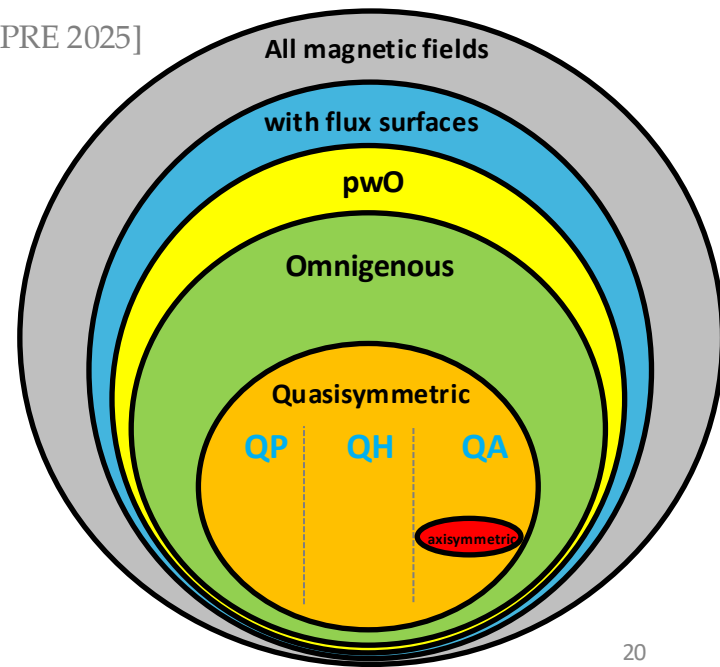
➤ *Piecewise omnigenous* **pwO** fields: a new class of fields with tokamak-like neoclassical transport

- $B = B_{max}$  is parallelogram-shaped (different from omnigenous)
- can be optimized for zero bootstrap current [Calvo et al PRE 2025]
- less constraining, potentially requires less shaping



[Velasco et al PRL 2024]

[Velasco et al PPCF 2025]

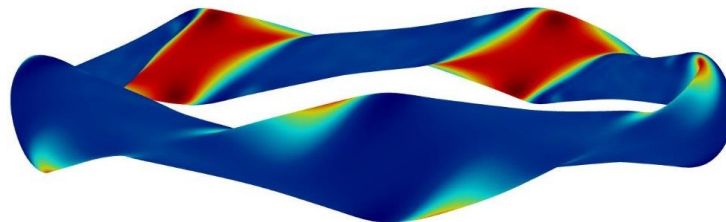
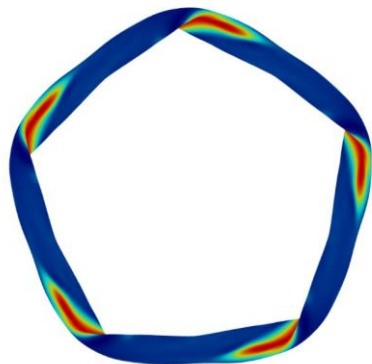




# A nearly pwO configuration made reactor relevant

**CIEMAT-pw1** fulfills a standard set of plasma physics criteria required for a stellarator reactor:

- Mercier MHD stability,
- $5/6 < \iota < 5/5 \Rightarrow$  no low order rationals, compatible with island divertor,
- Reduced neoclassical transport and bootstrap current ( $\Delta \iota_{LCFS} < 1\%$ ),
- Good fast ion confinement at reactor  $\beta$  ( $> 95\%$  alpha-heating, most losses for  $t > 0.01$  s),
- Reduced turbulence (maximum- $J$  property)

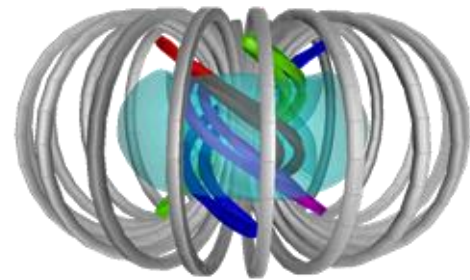
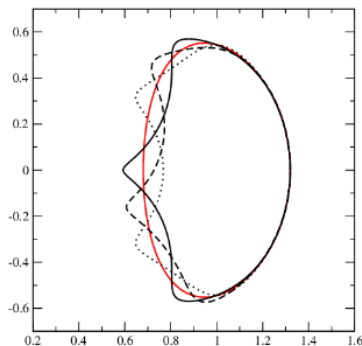
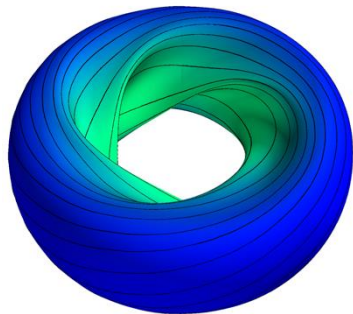


[Fernández-Pacheco, in preparation]



# Revived concept of tokamak-stellarator hybrid

- Optimized compact tokamak to preserve **good QA**, flux surfaces, and **external rotational transform**.
- Optimization with self-consistent bootstrap current and magnetic well.
- 4 “**banana coils**” coils of a **single type** (in addition to TF coils + PF coils).
- Also possible with modular coils.



[Henneberg & Plunk PRR 2024]  
[Schuett & Henneberg, PRR 2024]  
[Henneberg & Plunk et al PPCF 2025]  
[Schuett & Henneberg PPCF 2025]

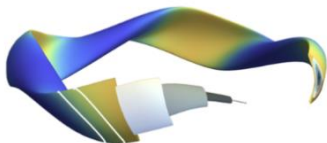




# Conclusion: a new ecosystem of tools & stellarator designs

- European collaboration TSVV-12 has pushed the frontier of stellarator optimization.
- Thanks to a excellent team with broad expertise, ~50 zoom meetings, strong links with other TSVVs and with the *Simons Collaboration* (5 in-person meetings in NY), and fantastic ACH support.
- The result is **66 journal publications** and a **new ecosystem** of modern **tools** and stellarator **designs**.

SQUID family



CIEMAT-QI family

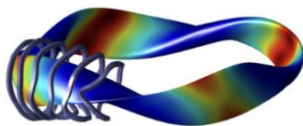
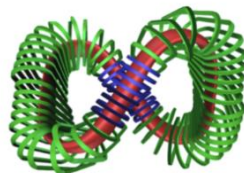
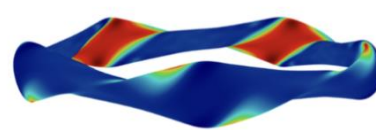


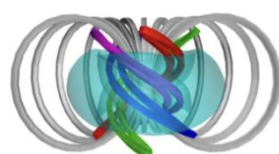
figure-8



pwO



hybrids



- TSVV-i will bring these (and other) designs to further maturity, incorporating latest tools & metrics.