

Investigations of plasma turbulence in the edge of HSX, LHD and W7-AS

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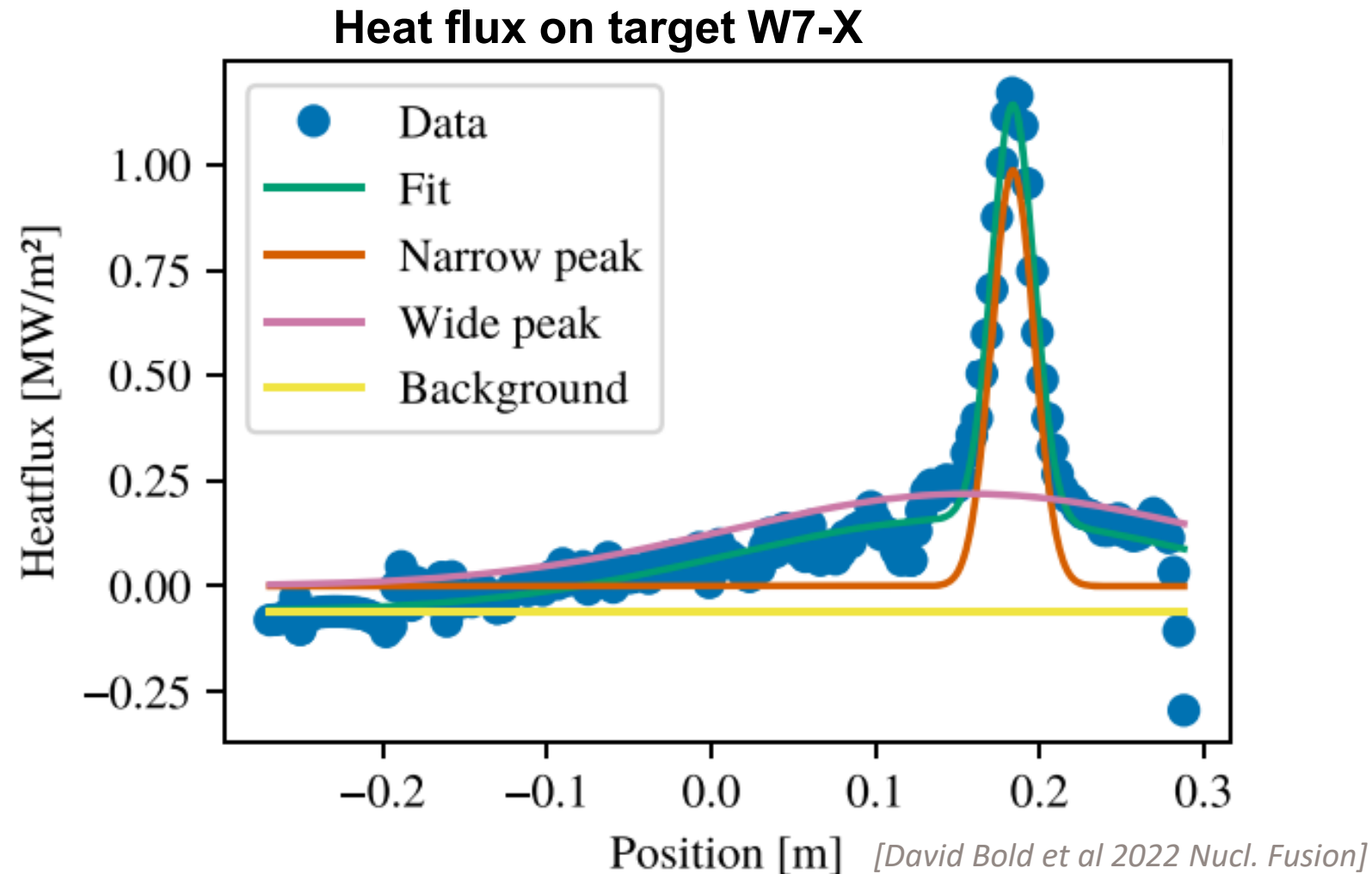
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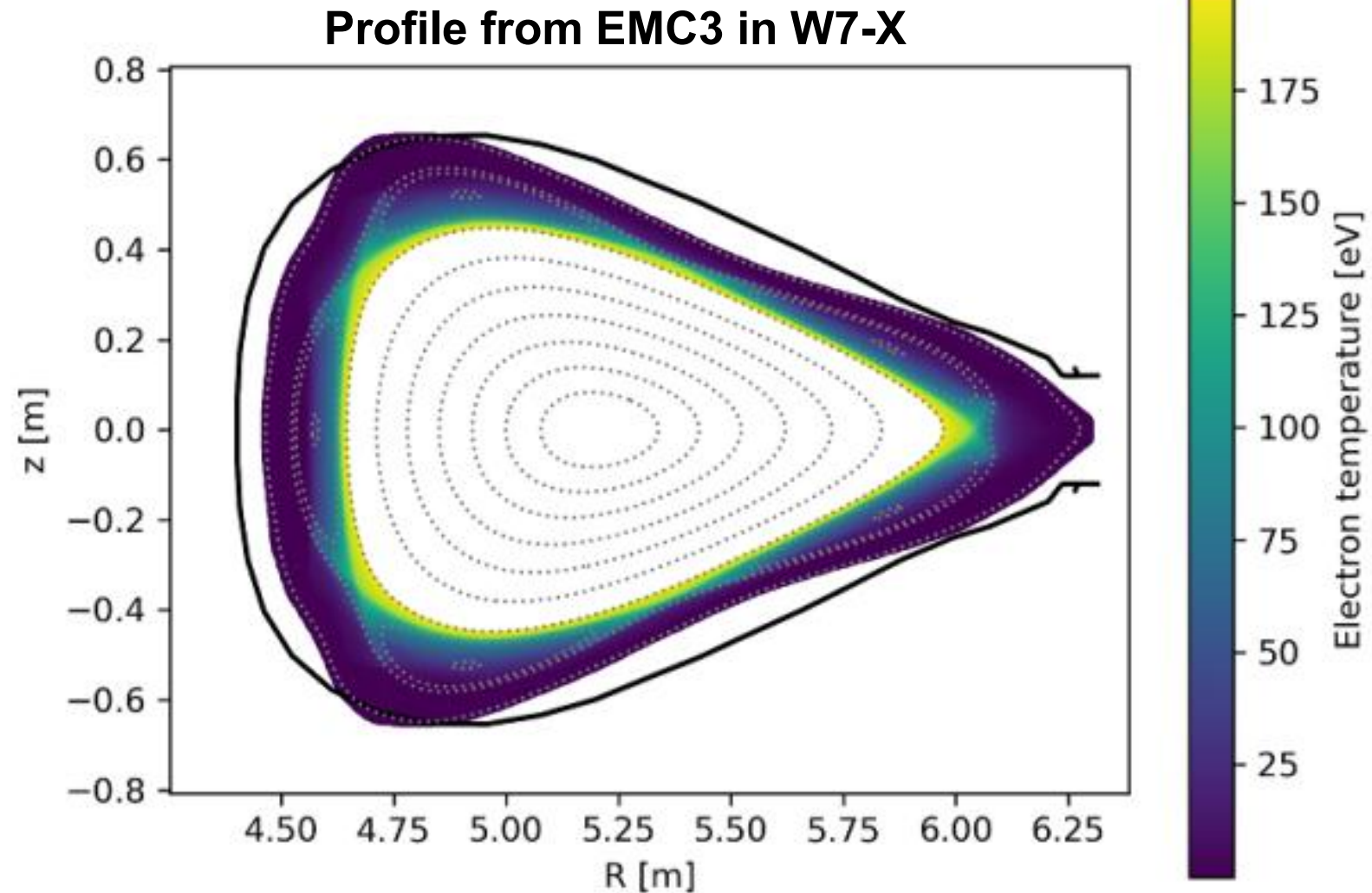
⁵Gauss Fusion GmbH, Garching bei München, Germany,

- Significant heat fluxes in the edge of stellarators
- Perpendicular transport influences peaks and spread of heat flux profile and SOL width
- The core plasma is constrained by boundary conditions established through edge turbulence



- Inclusion of turbulence positively affecting the divertor design
- Include edge turbulence metrics in the stellarator optimisation loop [M. J. Gerard et al, Nucl. Fusion 2023]

- Recent and past investigations are mainly based on **transport codes** [Feng, Y., et al. *Contributions to Plasma Physics* (2004)]
- Turbulent transport is modelled with **ad-hoc turbulent coefficients**
- Use turbulent codes to inform transport codes.

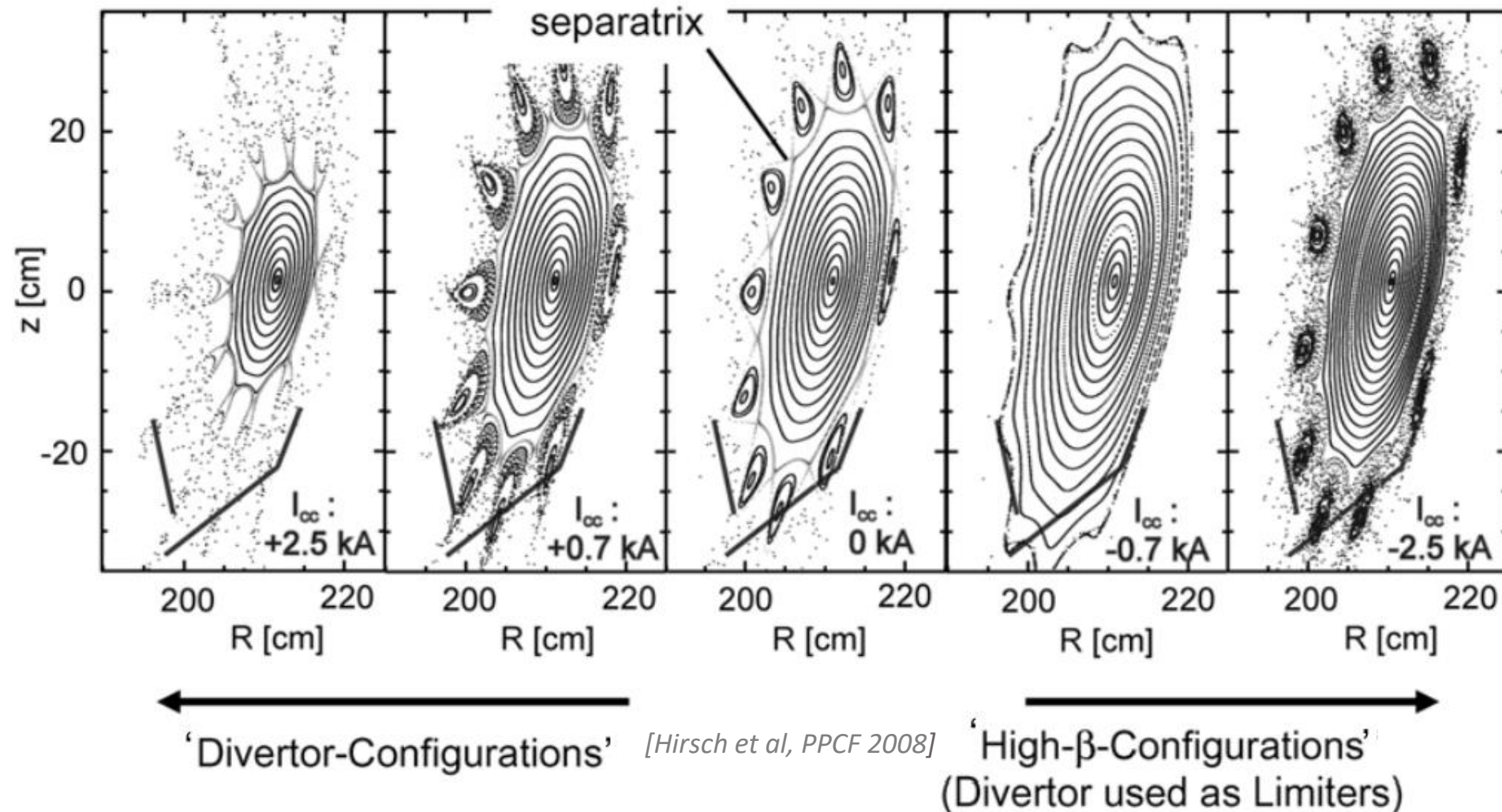


What makes the edge region challenging?

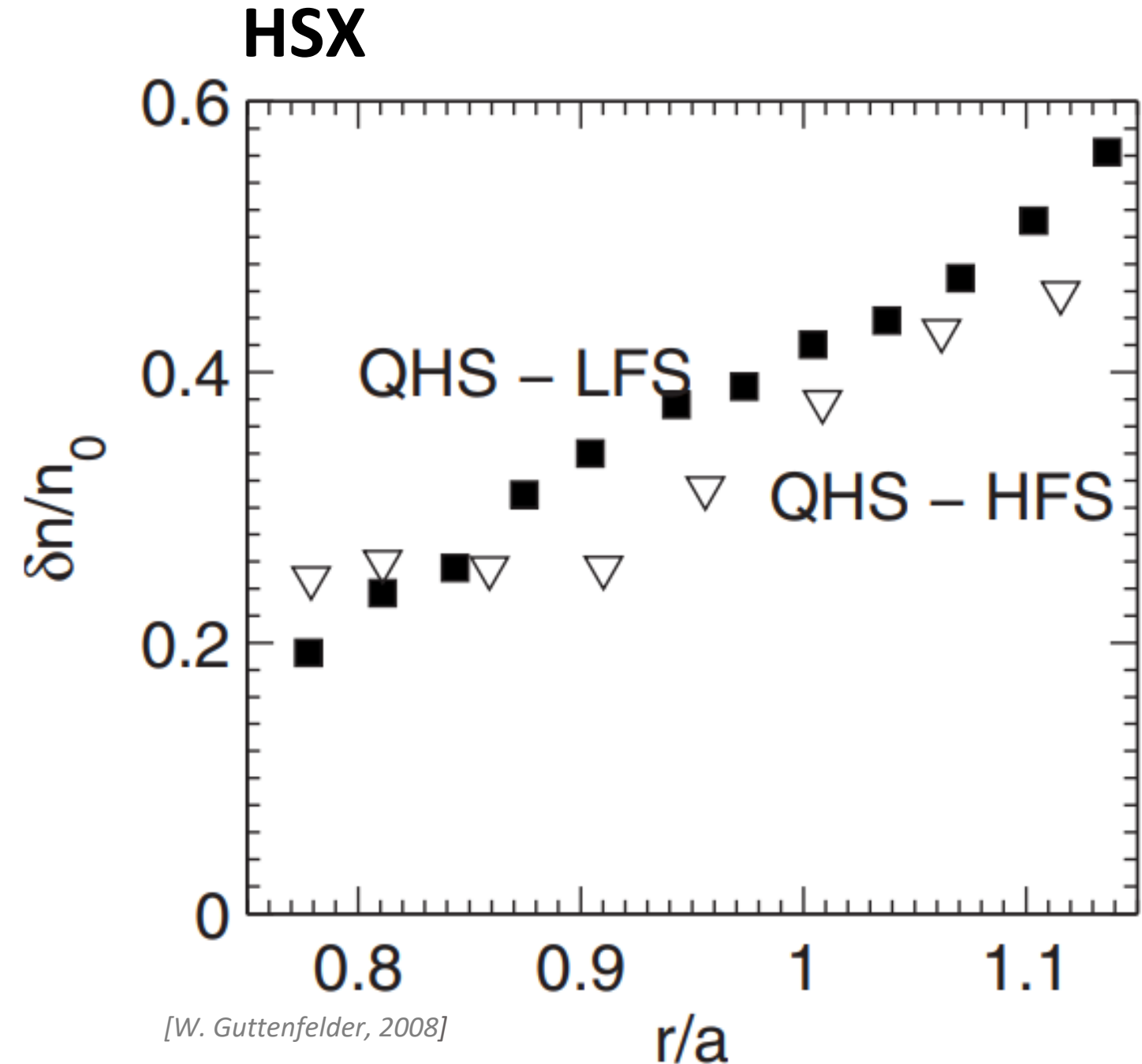
[David Bold et al 2024 Nucl. Fusion]

- **Edge complex magnetic structures:** magnetic islands, stochastic regions, X-points

W7-AS

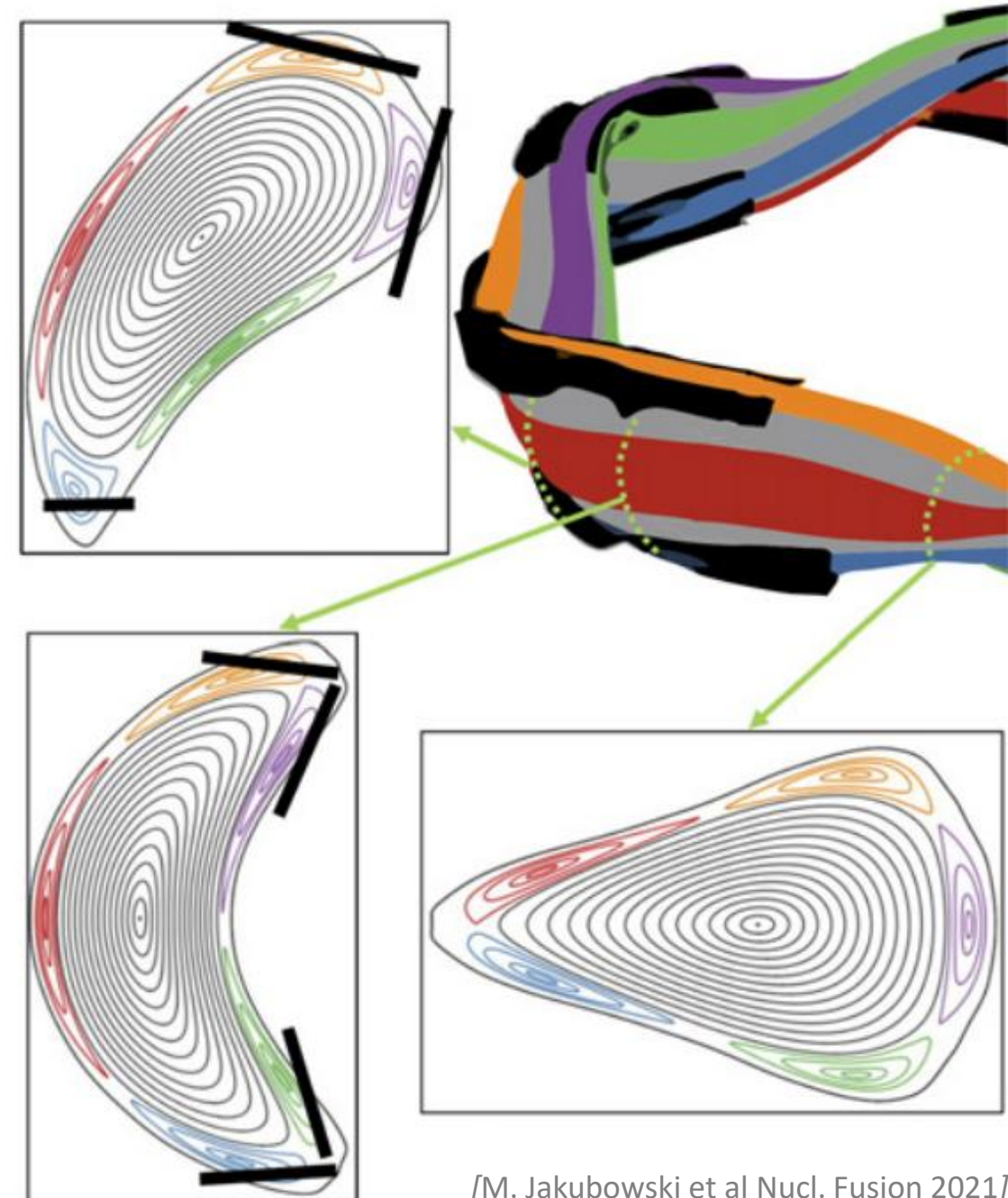


- **Edge complex magnetic structures:** magnetic islands, stochastic regions, X-points
- **Large fluctuations levels:** no separation between background and fluctuations



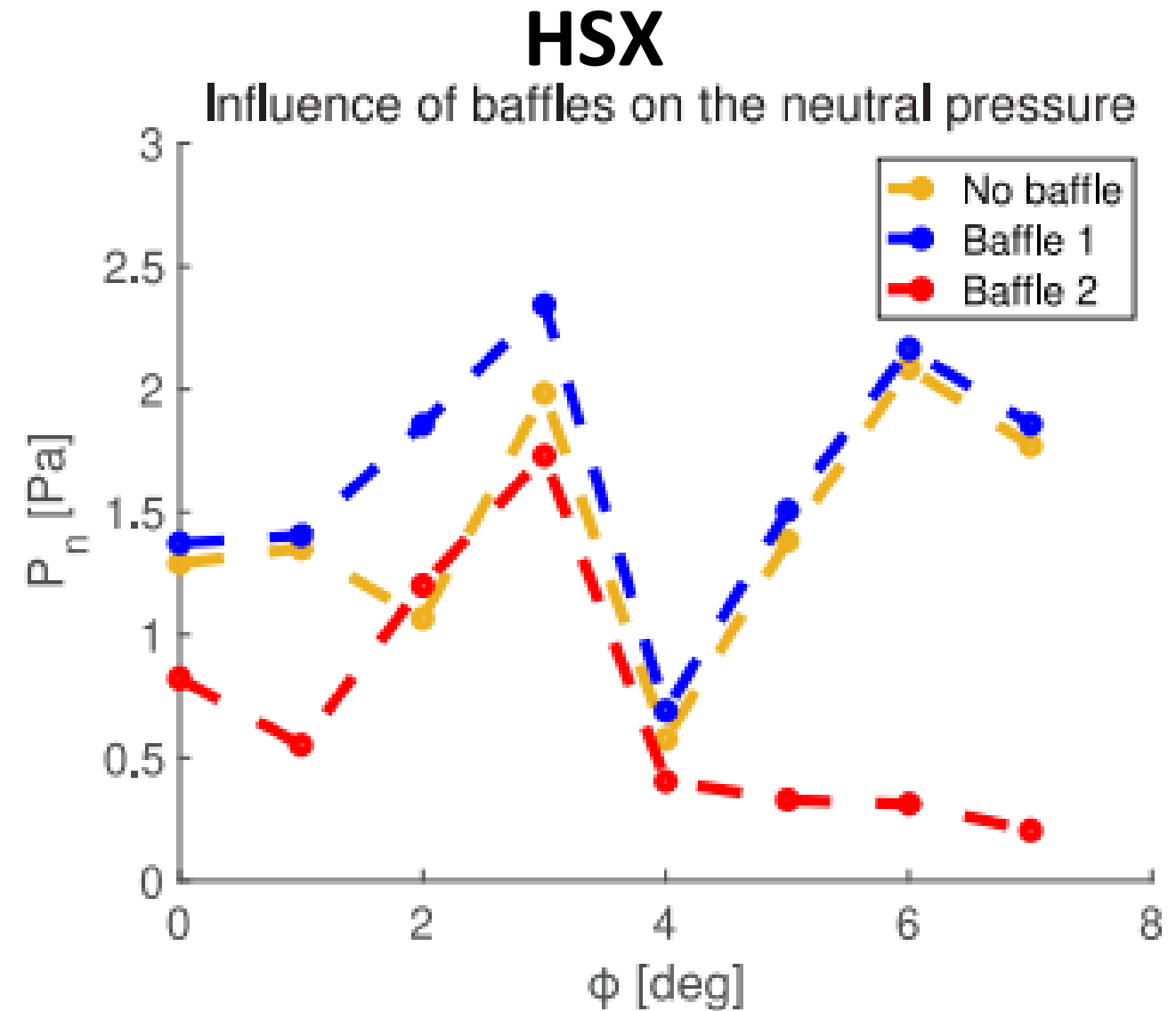
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W7-X

[M. Jakubowski et al Nucl. Fusion 2021]

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- **Complex diverting structures**
- **Neutral dynamics**



[D. Boeyaert et al *NI. Materials and Energy* 2025]

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Main challenges in the edge region

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Simulation requirements

- **Flexible discretization scheme with non-flux-aligned coordinates**
- **No separation of equilibrium and fluctuating quantities**
- **Flux-driven:** sources generate gradients for turbulence driving
- **Global:** simulate the entire volume
- **Magnetic pre-sheath boundary conditions**
- **Coupling plasma and neutrals**

Drift-reduced Braginskii equations to simulate the boundary plasma turbulence

- Plasma edge is **highly collisional** $\rho_{L,i} k_{\perp} \ll 1$, $\lambda_{mfp} \ll L_{\parallel}$ use **two-fluid Braginskii model**
[S. I. Braginskii. *Reviews of Plasma Physics*, 1965]

- Turbulence time scale longer than Ω_{ci}^{-1} , **drift-reduced approximation** [A. Zeiler, (1999)]

$$\mathbf{V}_e = \mathbf{V}_{E \times B} + \mathbf{V}_{de} + V_{\parallel e} \mathbf{b}, \quad \mathbf{V}_i = \mathbf{V}_{E \times B} + \mathbf{V}_{di} + \mathbf{V}_{pol} + V_{\parallel e} \mathbf{b}$$

- Quasi-neutrality:** $n_e \sim n_i \sim n$,
$$\frac{\partial n}{\partial t} + \underbrace{\nabla \cdot \Gamma_{E \times B}}_{\text{E} \times \text{B convection}} + \underbrace{\nabla \cdot \Gamma_{di}}_{\text{diamagnetic convection}} + \overbrace{\nabla \cdot \Gamma_{pol}}^{\text{polarization convection}} + \underbrace{\nabla_{\parallel} \Gamma_{\parallel}}_{\text{parallel flow}} = \underbrace{\mathcal{S}_n}_{\text{source}}.$$

- Electrostatic perpendicular electric field:** $\mathbf{E}_{\perp} = -\nabla_{\perp} \Phi$ solved with **Poisson**

- Similar equations for $V_{\parallel,e}, V_{\parallel,i}, T_e, T_i$ given by **momentum and energy conservation**

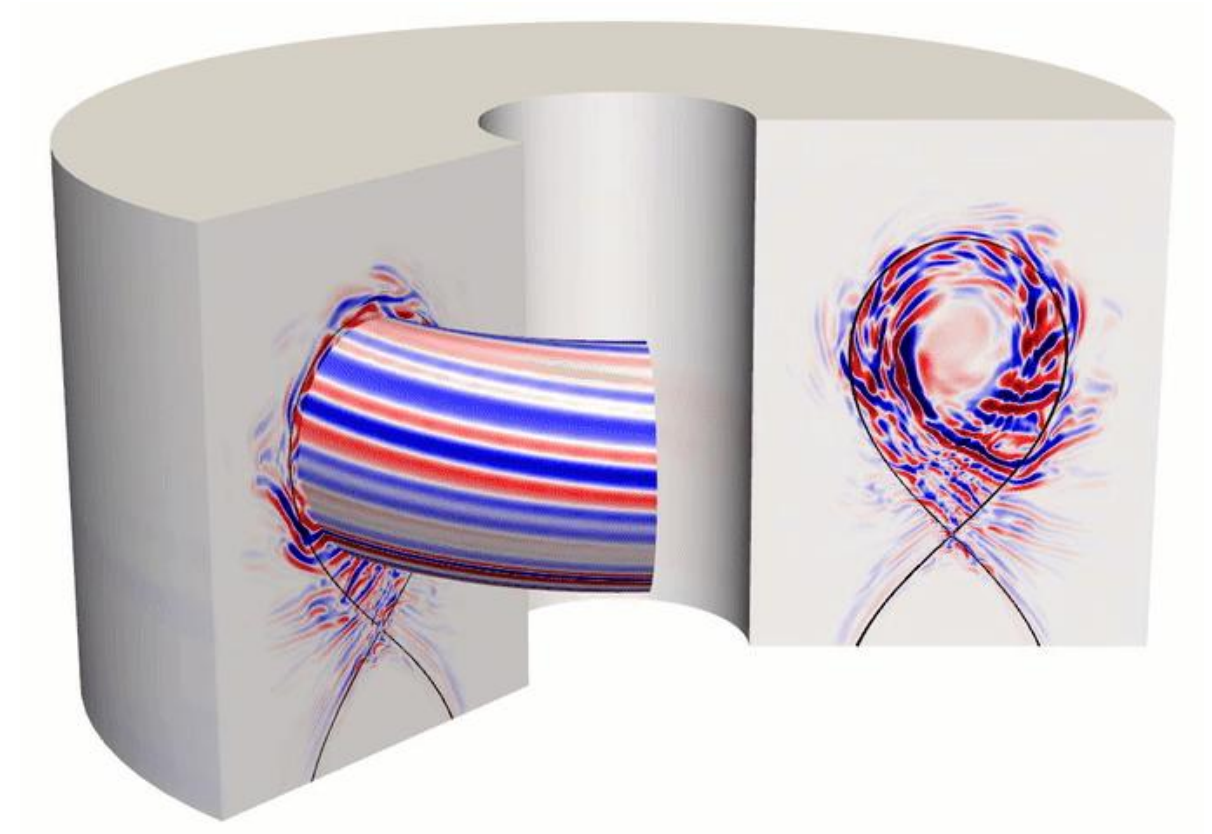
- Charge conservation,** $\nabla \cdot (\mathbf{J}_{\parallel} + \mathbf{J}_{di} + \mathbf{J}_{pol}) = 0$ **+ Neutral dynamics** (not considered in the present work) [D. Mancini et al *Nucl. Fusion* 2023]

- GBS is a **two fluid, global, flux-driven code** to simulate **turbulence** evolving such equations in time in a **non-flux-aligned grid**
- **Coupled with a kinetic model for neutral dynamics**

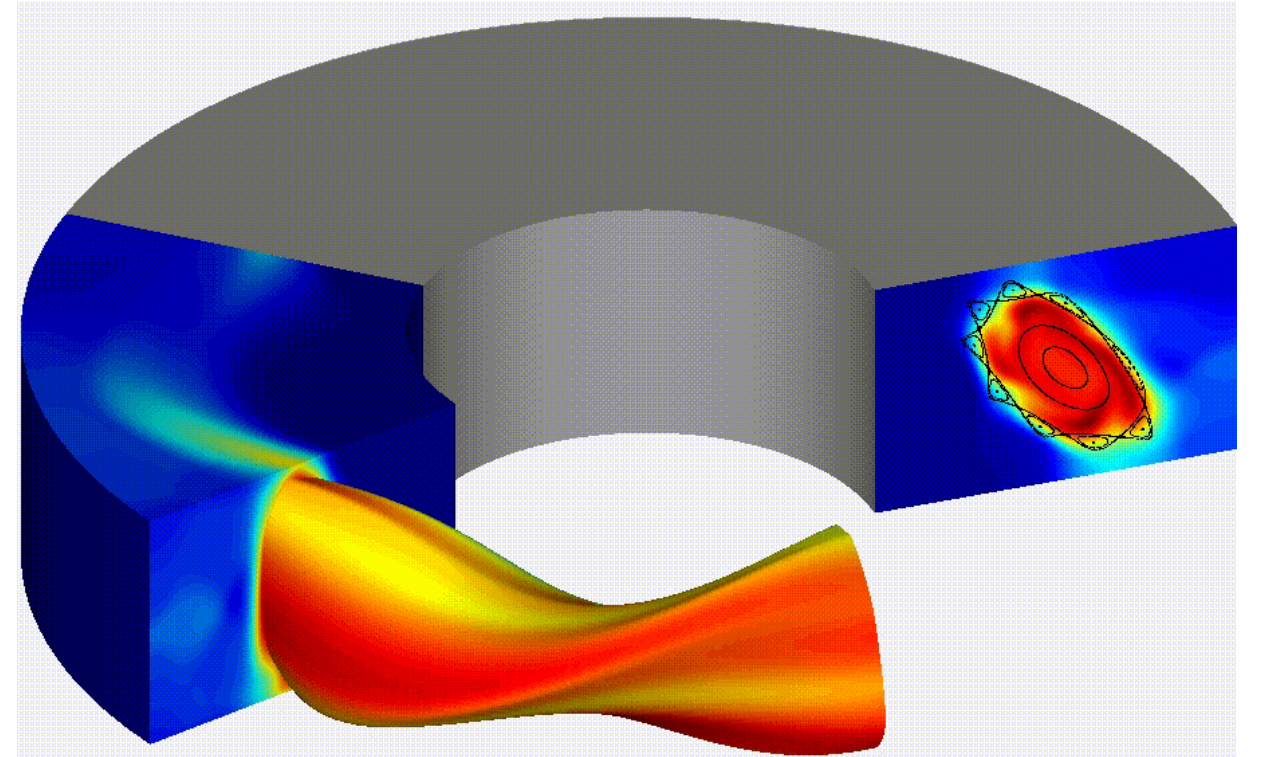
[D. Mancini et al Nucl. Fusion 2023]

TCV-like tokamak

[B. De Lucca et al EFTC 2025]



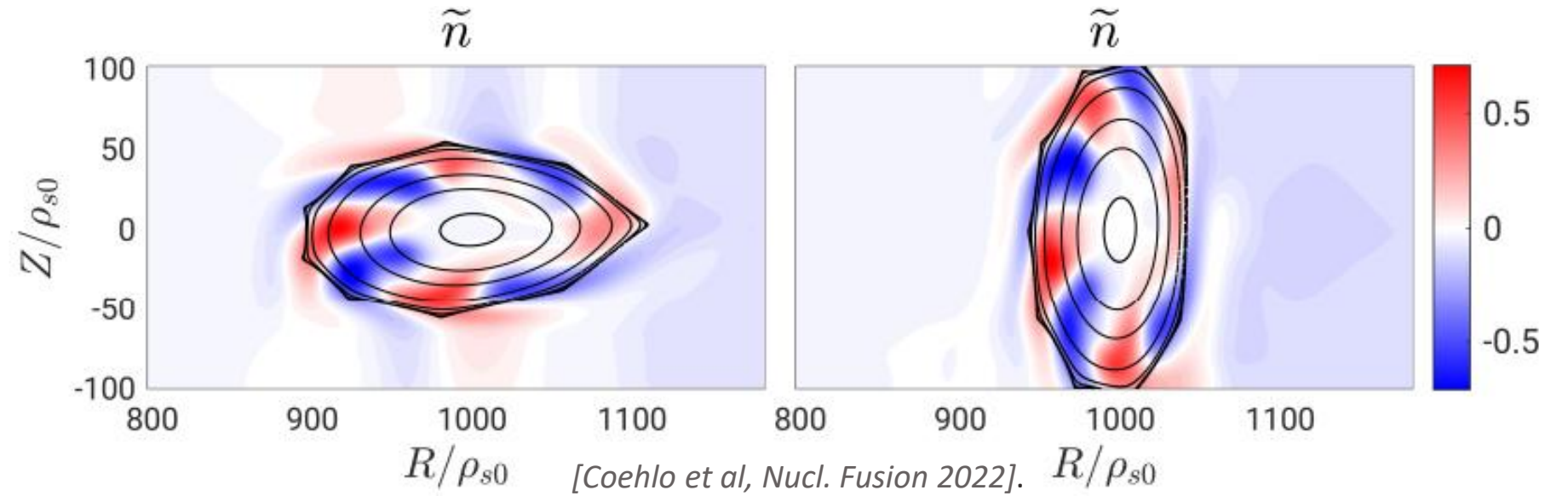
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[D. Mancini et al Nucl. Fusion 2023]
- GBS used in the past decade to simulate tokamaks *[Ricci et al. PPCF 2012, Giacomini et al. JCP 2022]*; **now it can simulate turbulence in 3D magnetic fields** *[Coelho et al, Nucl. Fusion 2024]*.



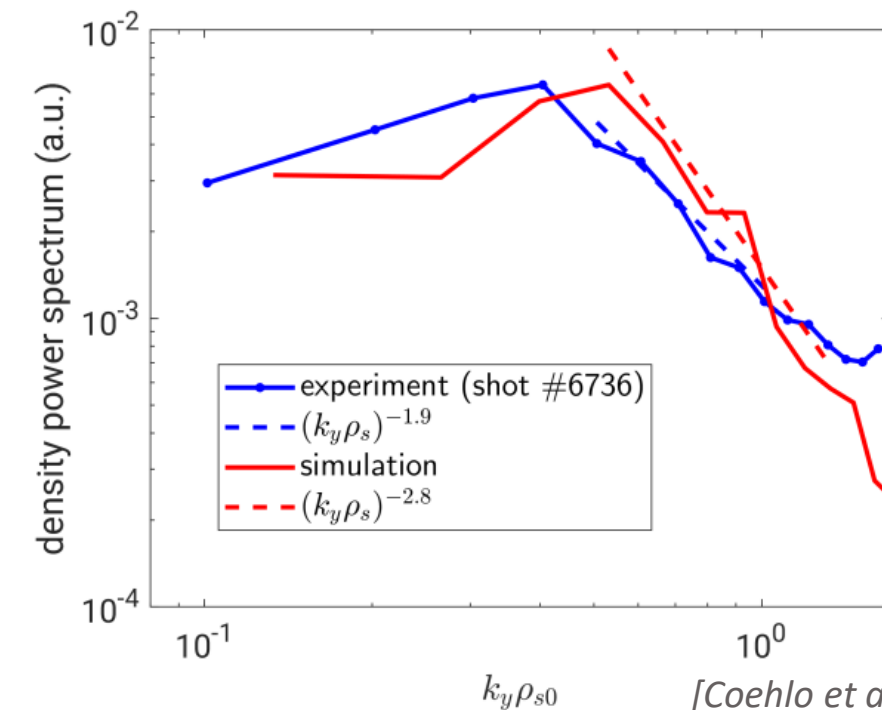
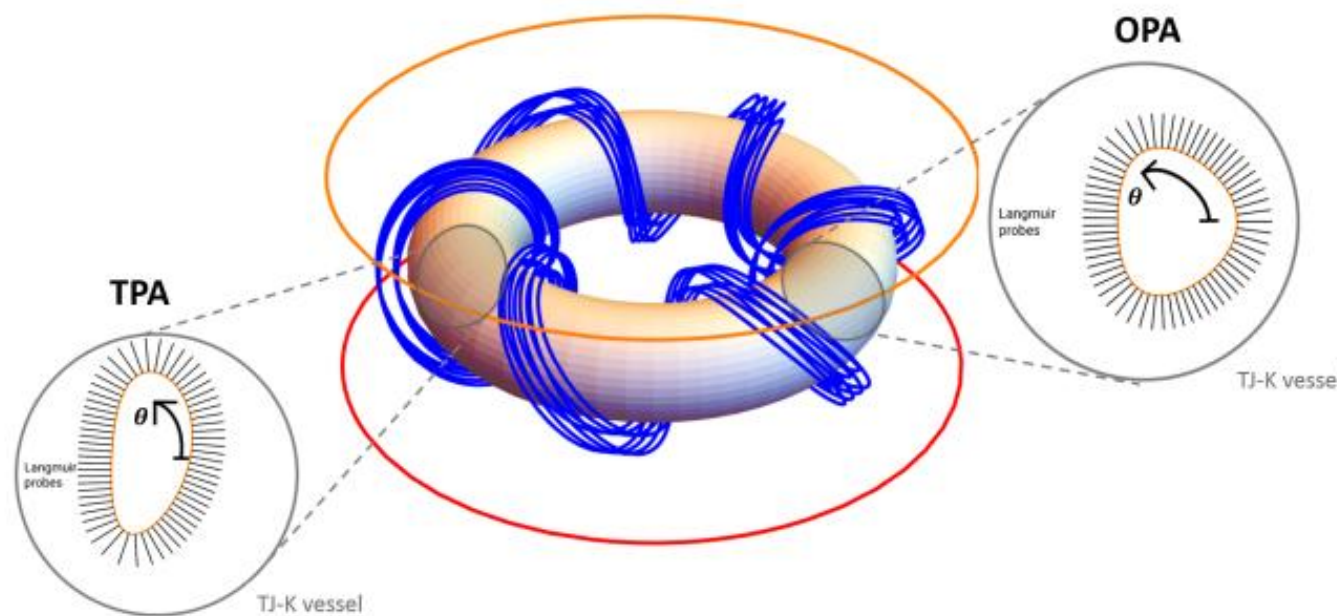
[Coelho et al, Nucl. Fusion 2024]

First-principles edge transport pioneered by the GBS group

- First global flux driven simulation of a toy model stellarator

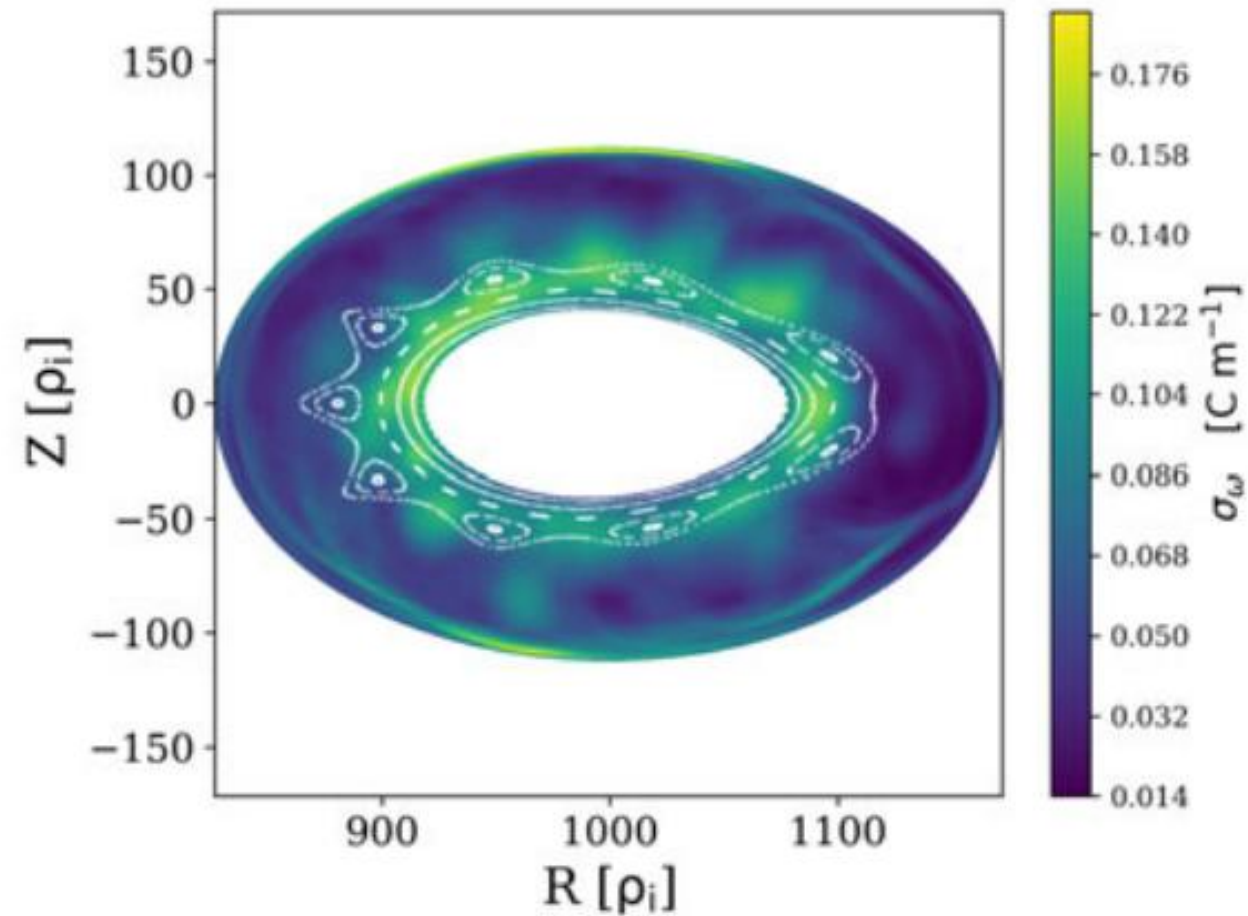


- Successful global quantitative validation in TJK



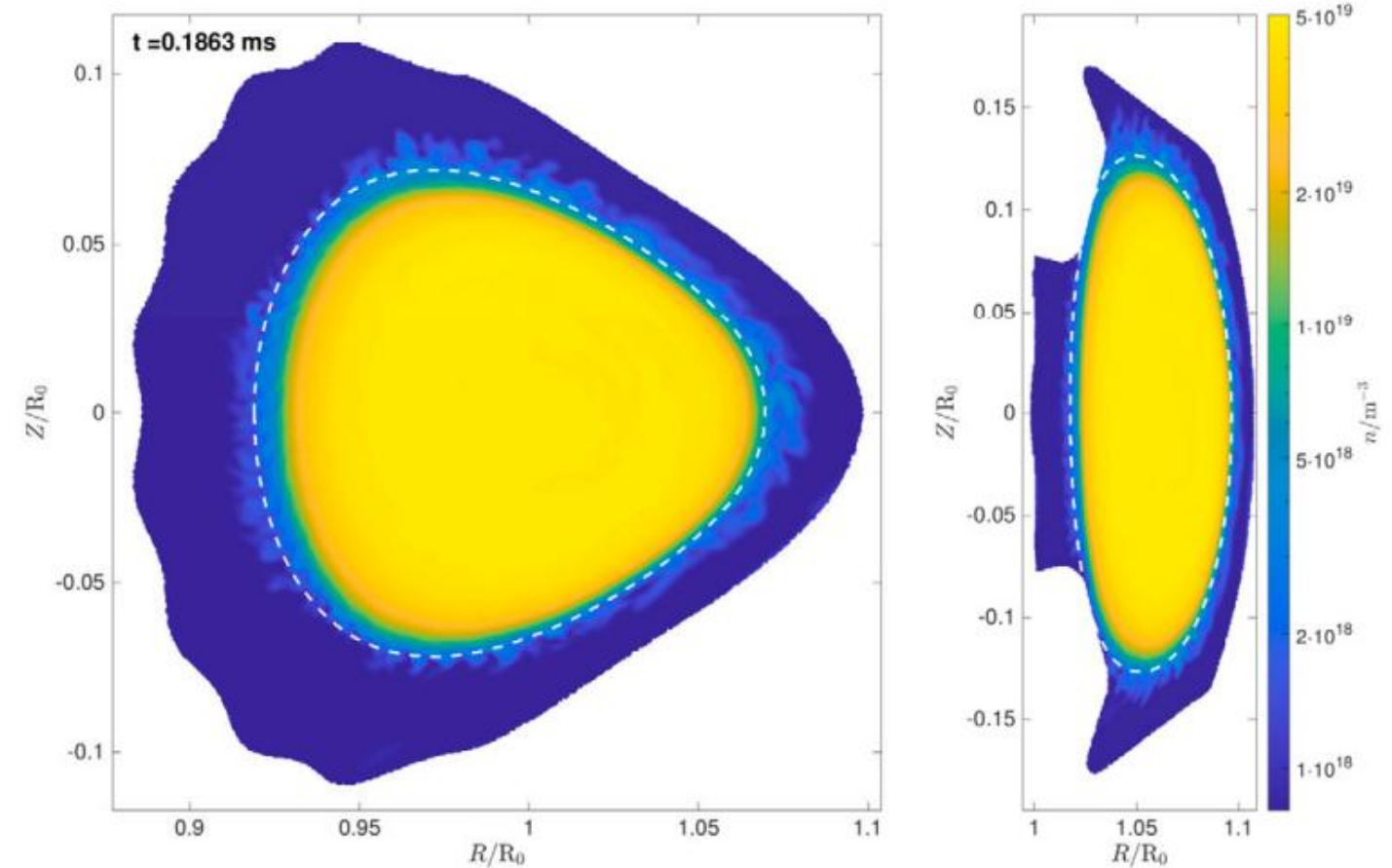
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 - Isothermal fluid model in BOUT++



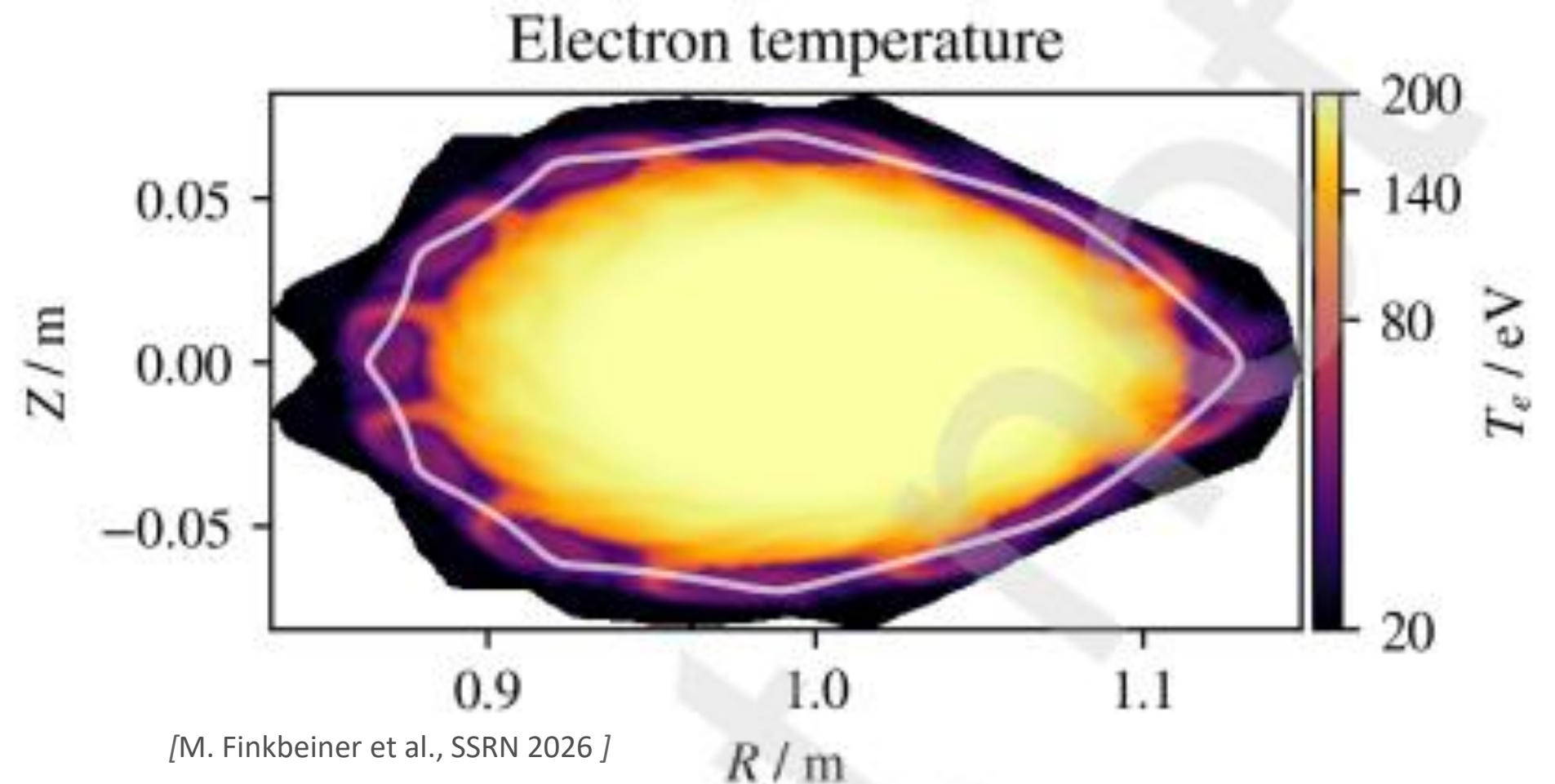
[B Shanahan et. al., JPP 2024]

- Other groups followed with different approaches
 - Isothermal fluid model in BOUT++
 - Plasma relaxation fluid model GRILLIX

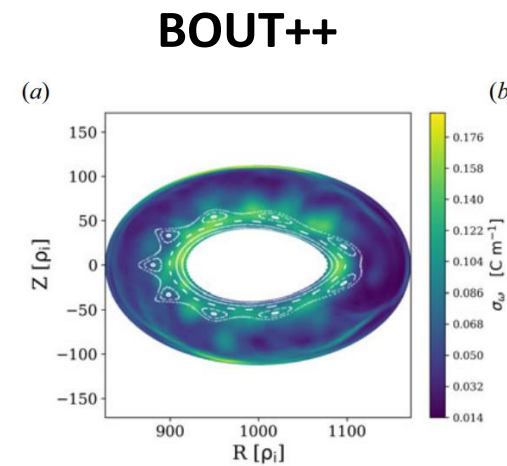


[A. Stegmeir et. al, JCP 2025]

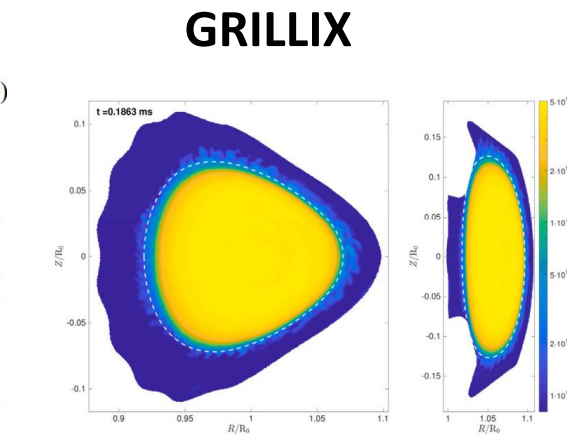
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 - Plasma relaxation kinetic model GENE-X



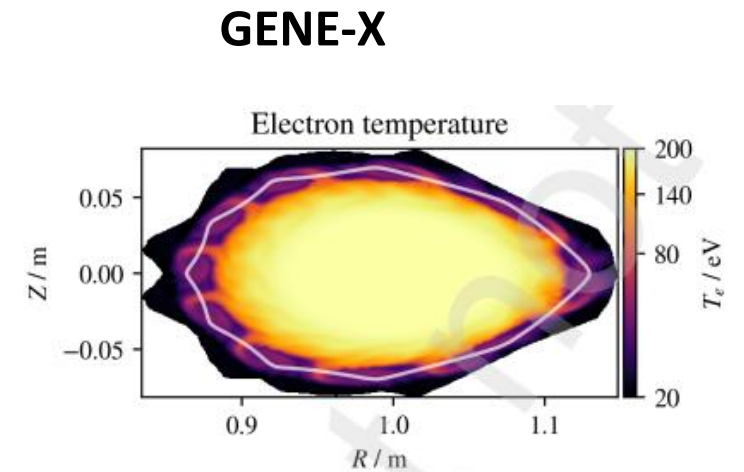
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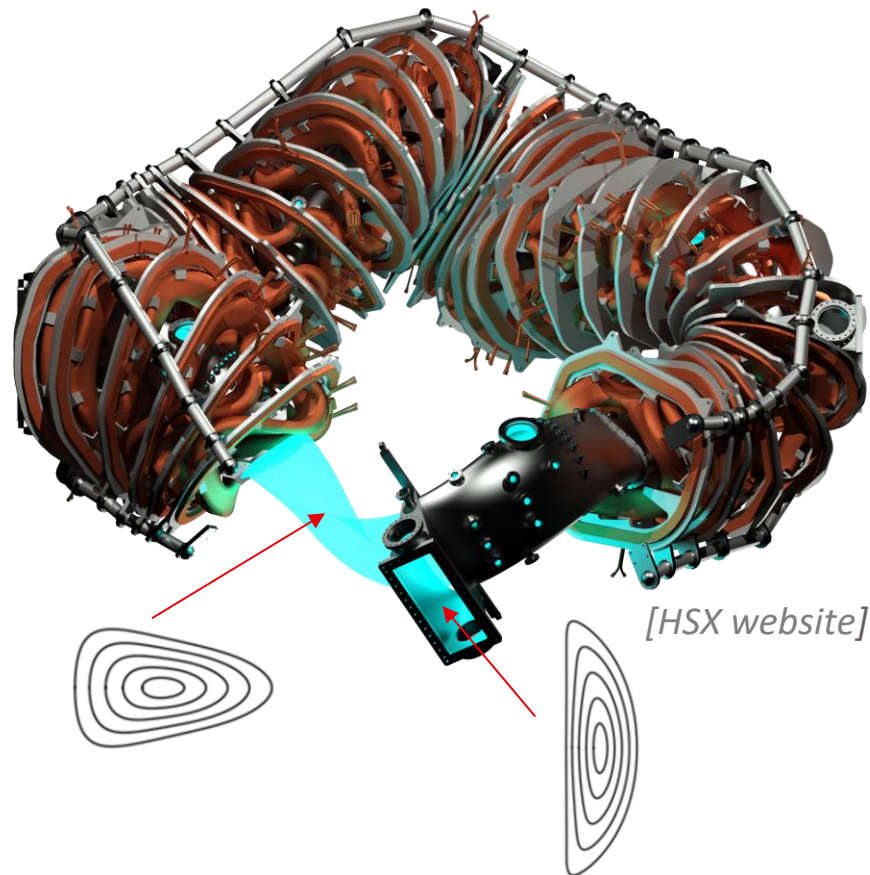
[A. Stegmeir et. al, JCP 2025]



[M. Finkbeiner et al., SSRN 2026]

- Here we examine and validate fusion-relevant machines, to understand the impact of three-dimensional geometries on edge turbulence

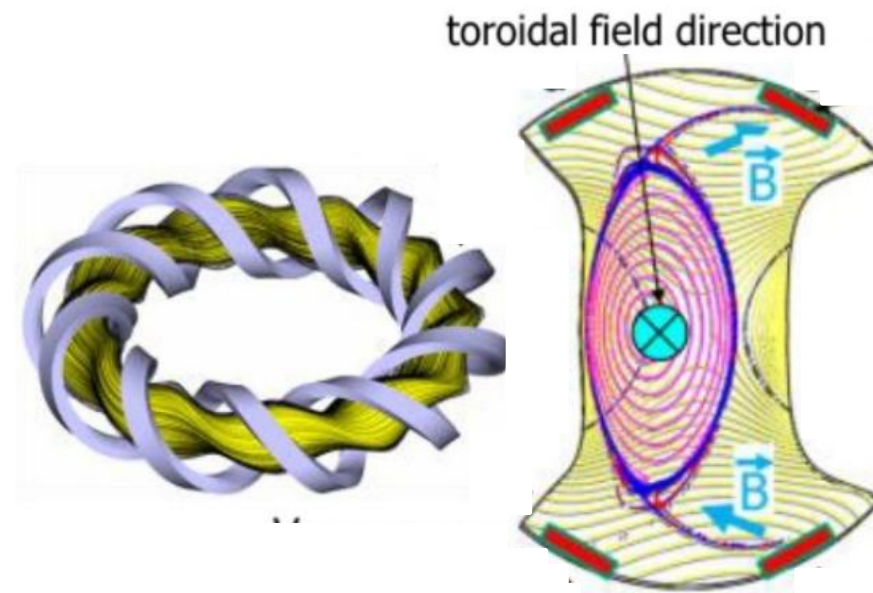
HSX



[HSX website]

Only **quasi-helically** stellarator in the world

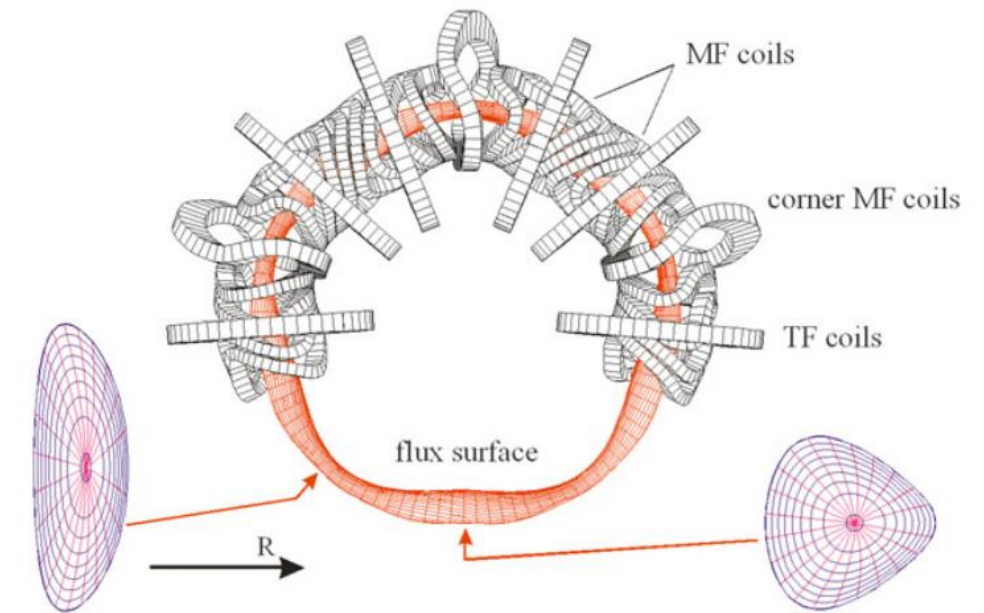
LHD



[S Masuzaki et al. Nuclear Materials and Energy 2019]

Largest **heliotron** stellarator in the world
Stochastic layer, X-point divertors

W7-AS

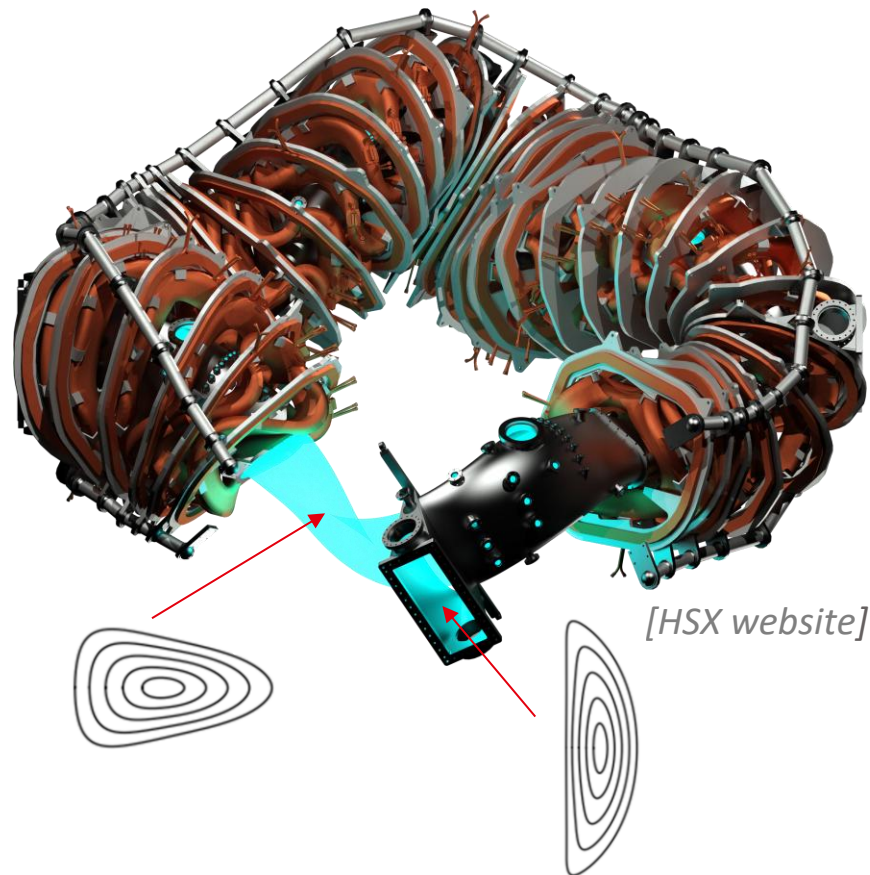


[Hirsch et al, PPCF 2008]

Predecessor of **W7-X** sharing many features
but smaller in size

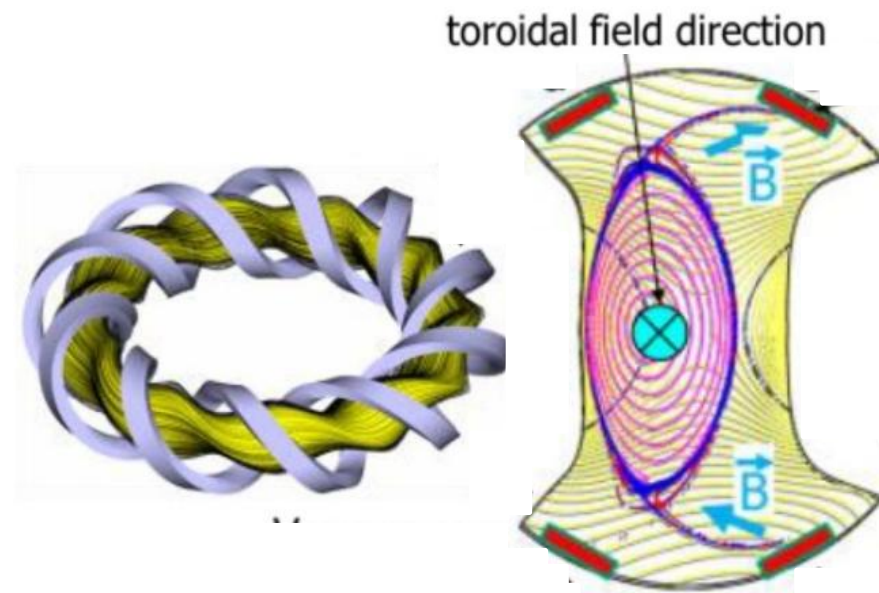
Auxiliary coil currents allowed **great flexibility**
in the edge topology

HSX



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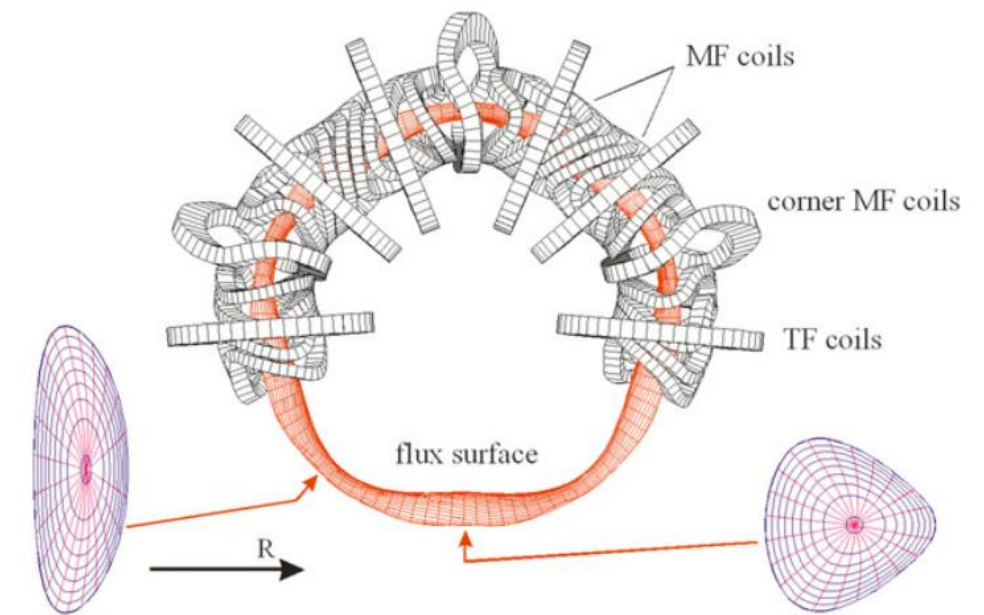
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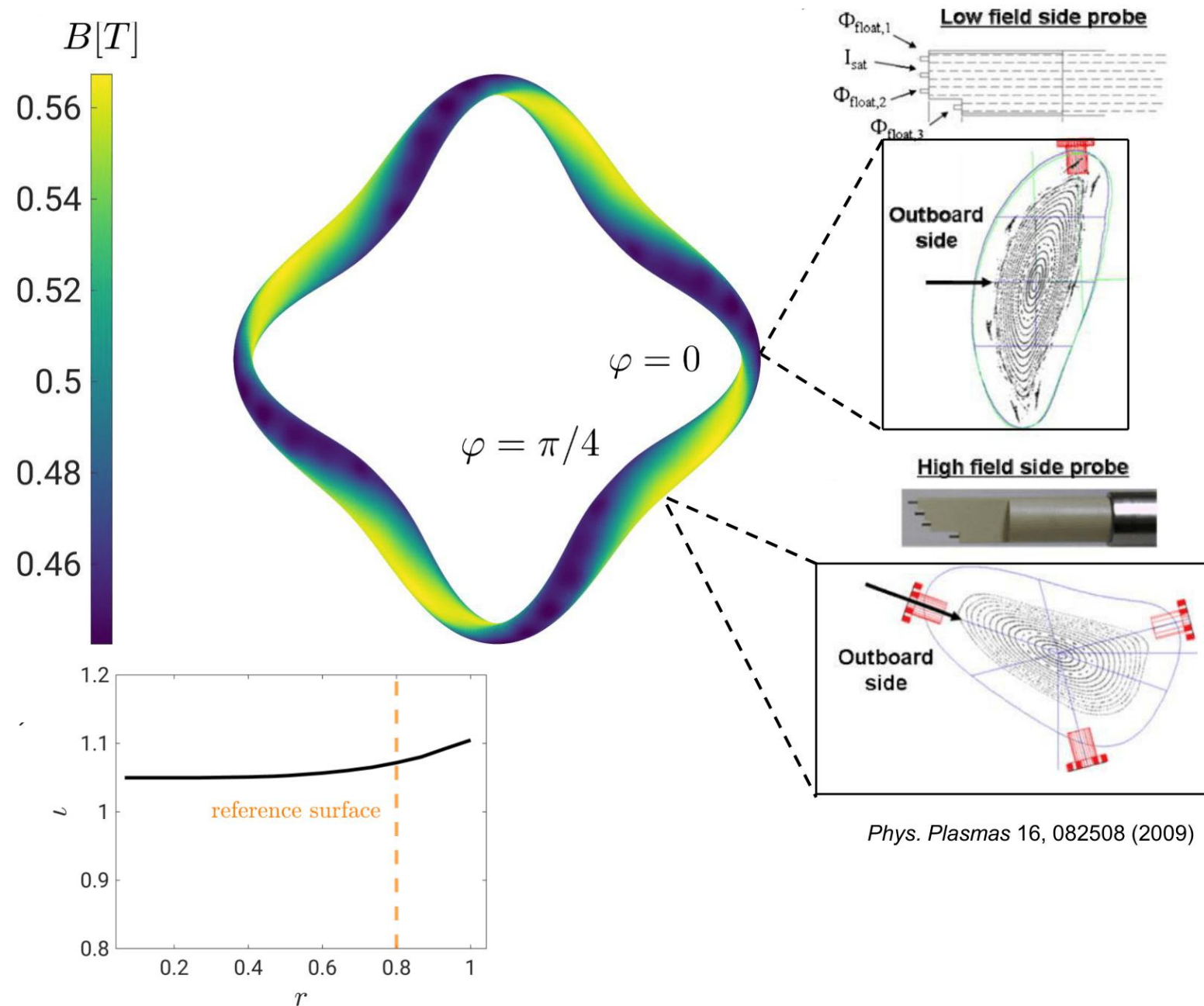
W7-AS



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- Simulate full-size machine with comparison to experiments done by W. Guttenfelder 2009
- Langmuir probes and Thomson scattering diagnostic
- Turbulent properties compared at the **reference surface**

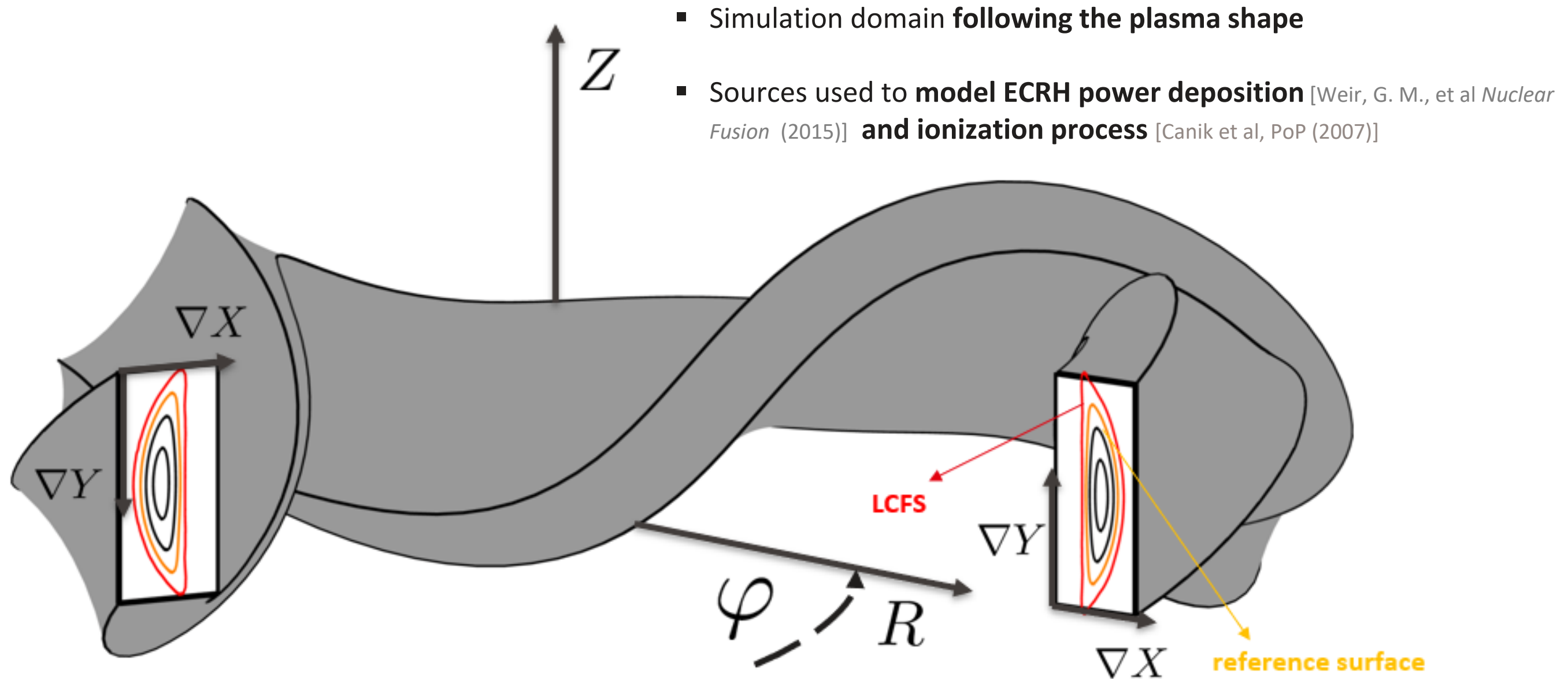
$$B_0 = 0.5 T$$

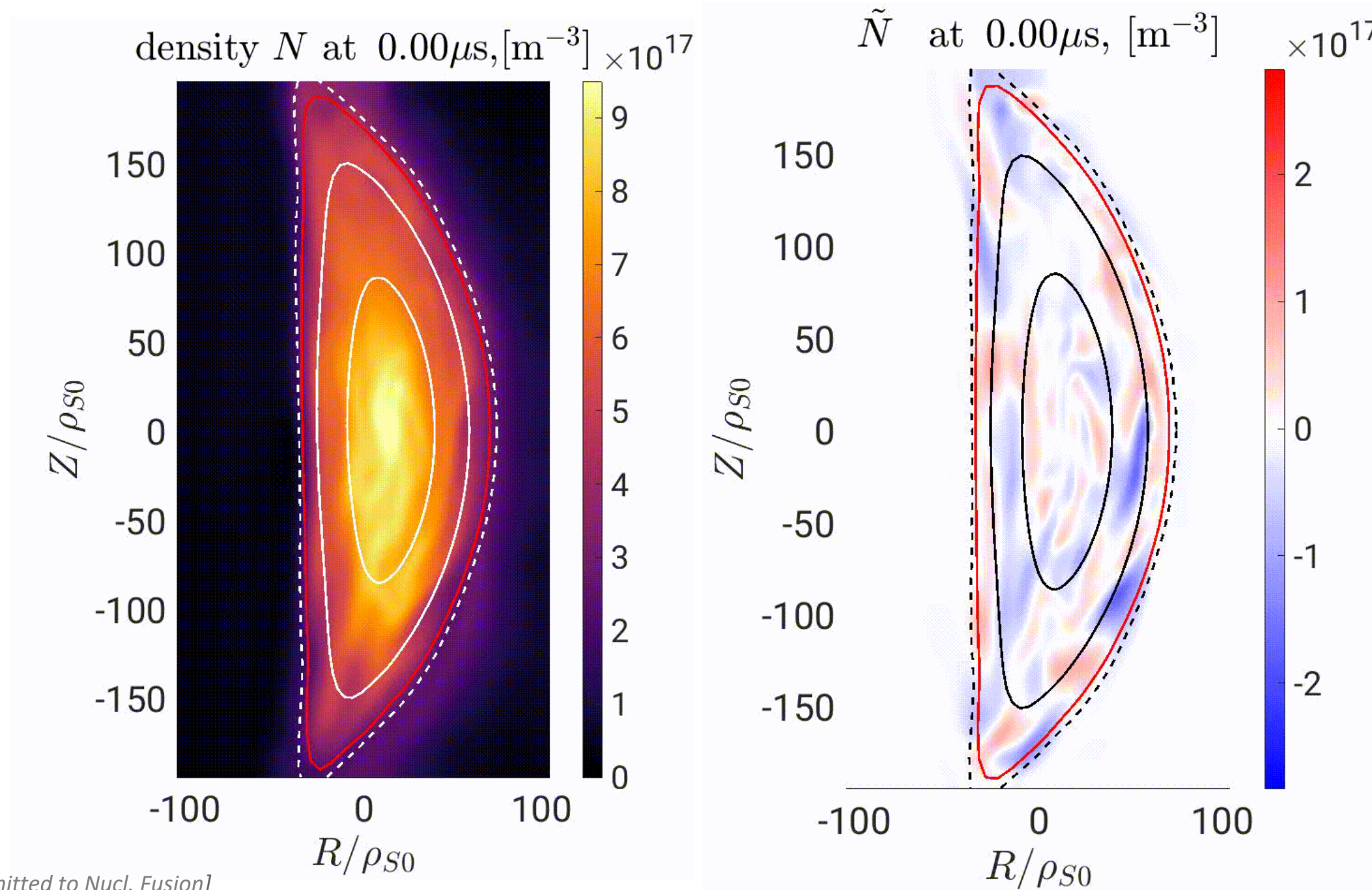
$$n_0 = 10^{18} m^{-3}$$

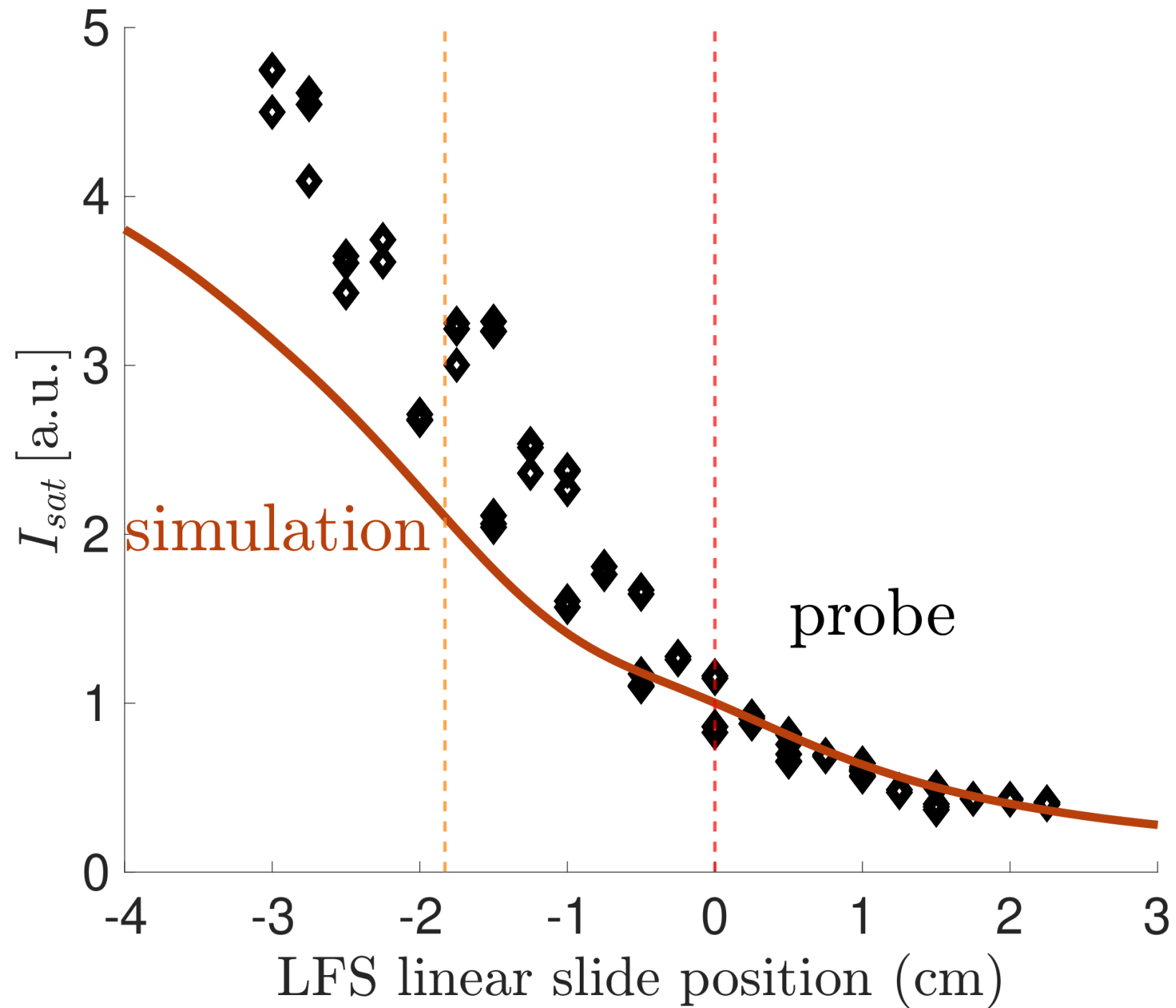
$$T_e = 40 eV$$

$$T_i = 20 eV$$

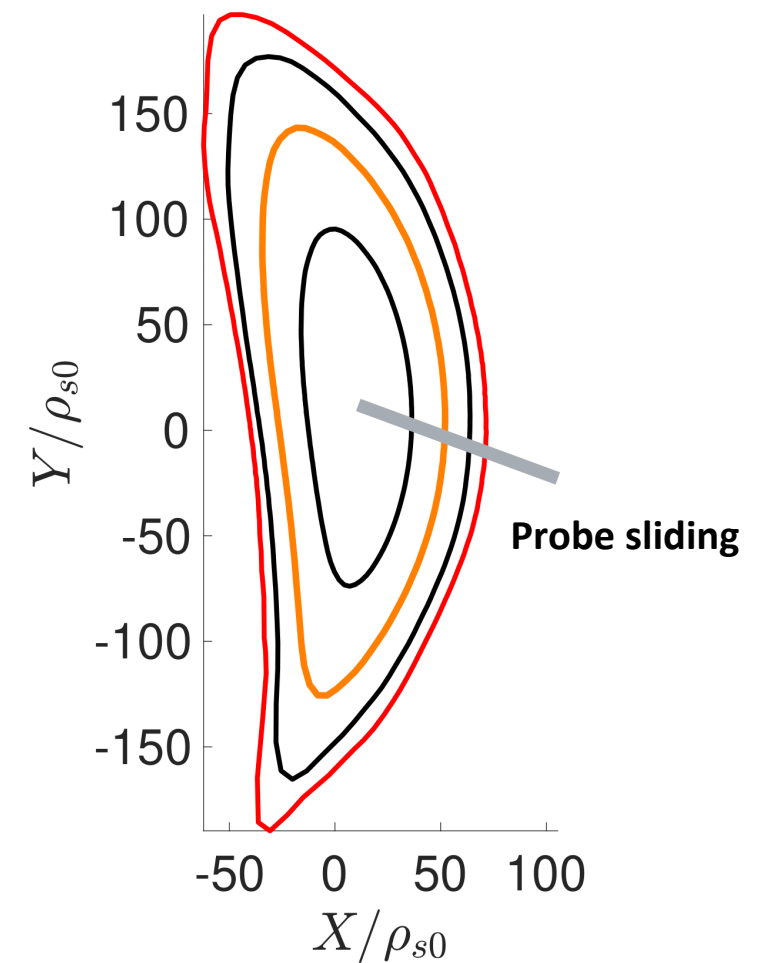
Phys. Plasmas 16, 082508 (2009)

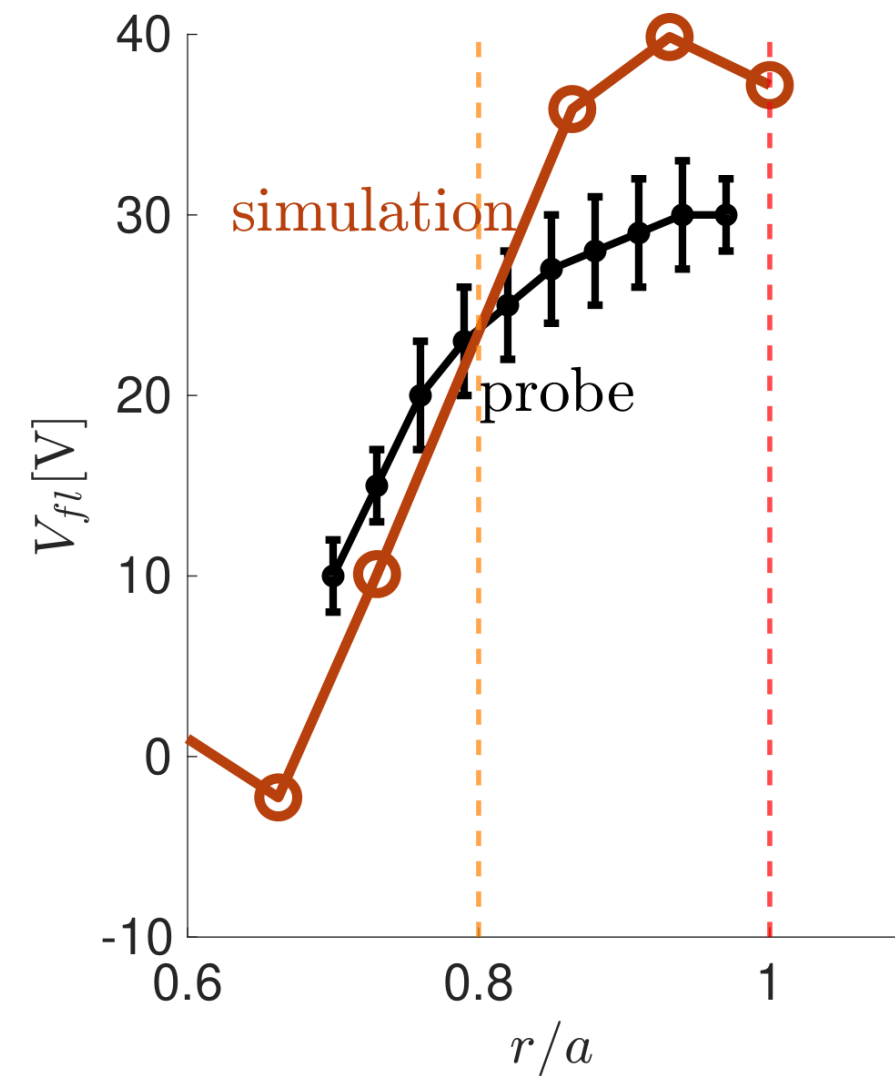
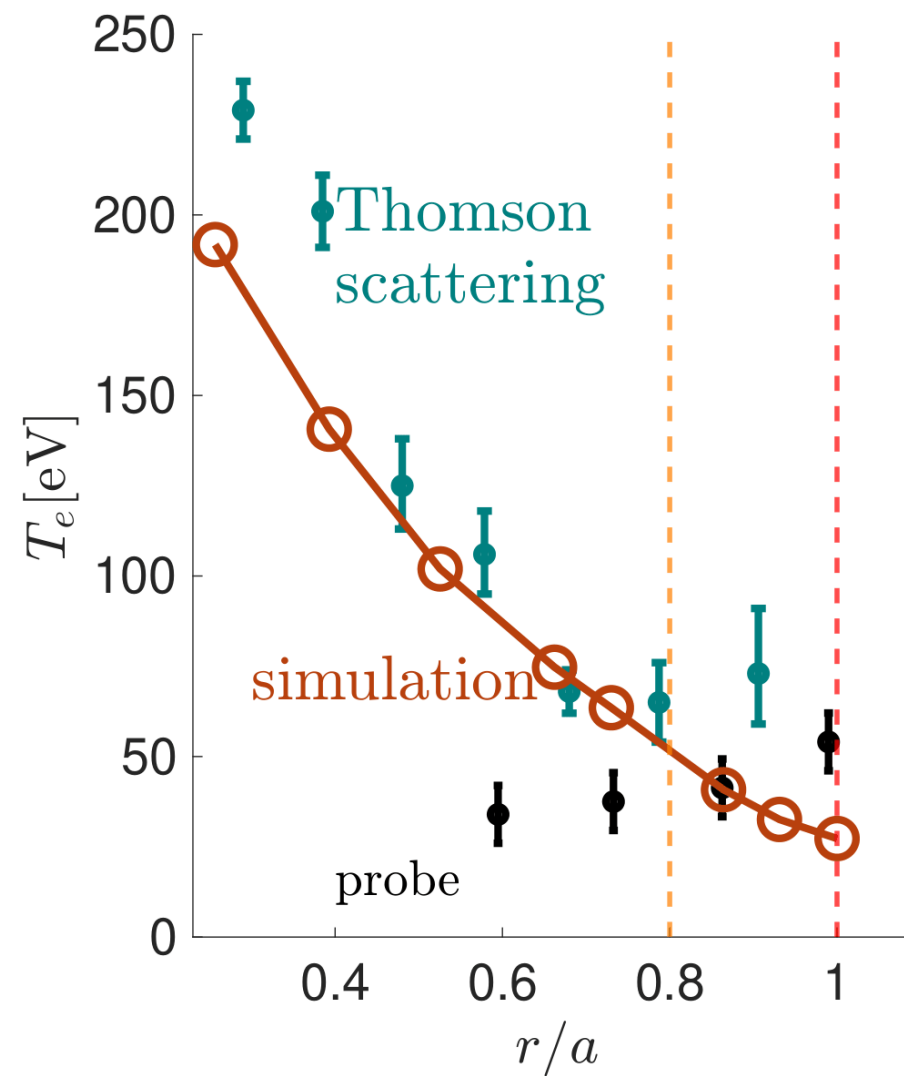
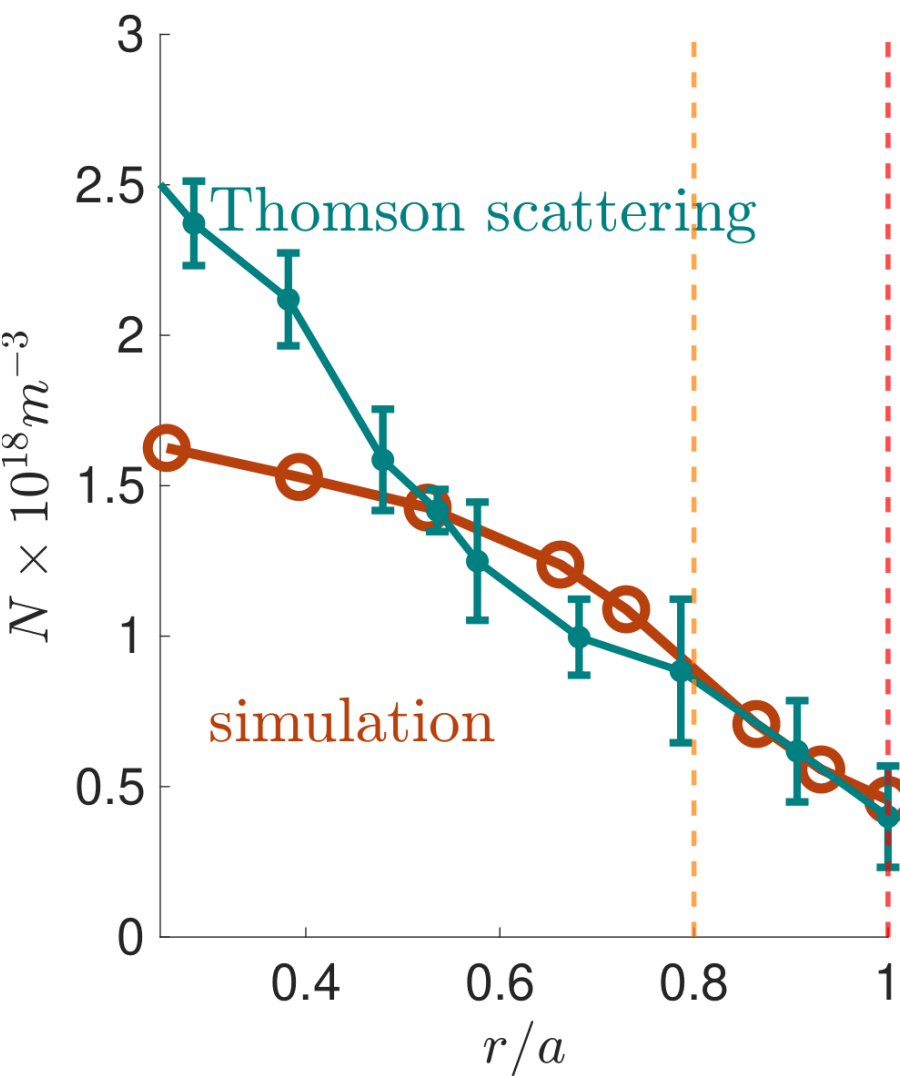




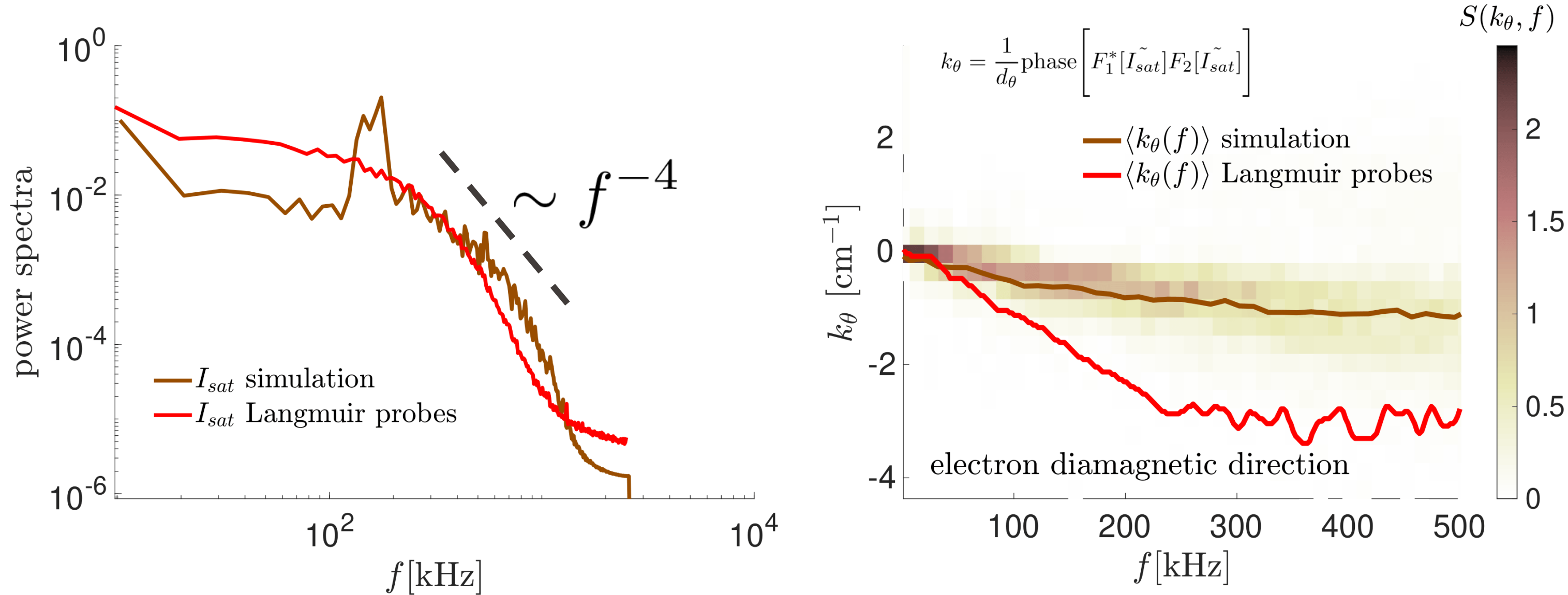


- Scaling length in the **SOL** ~ 2 cm

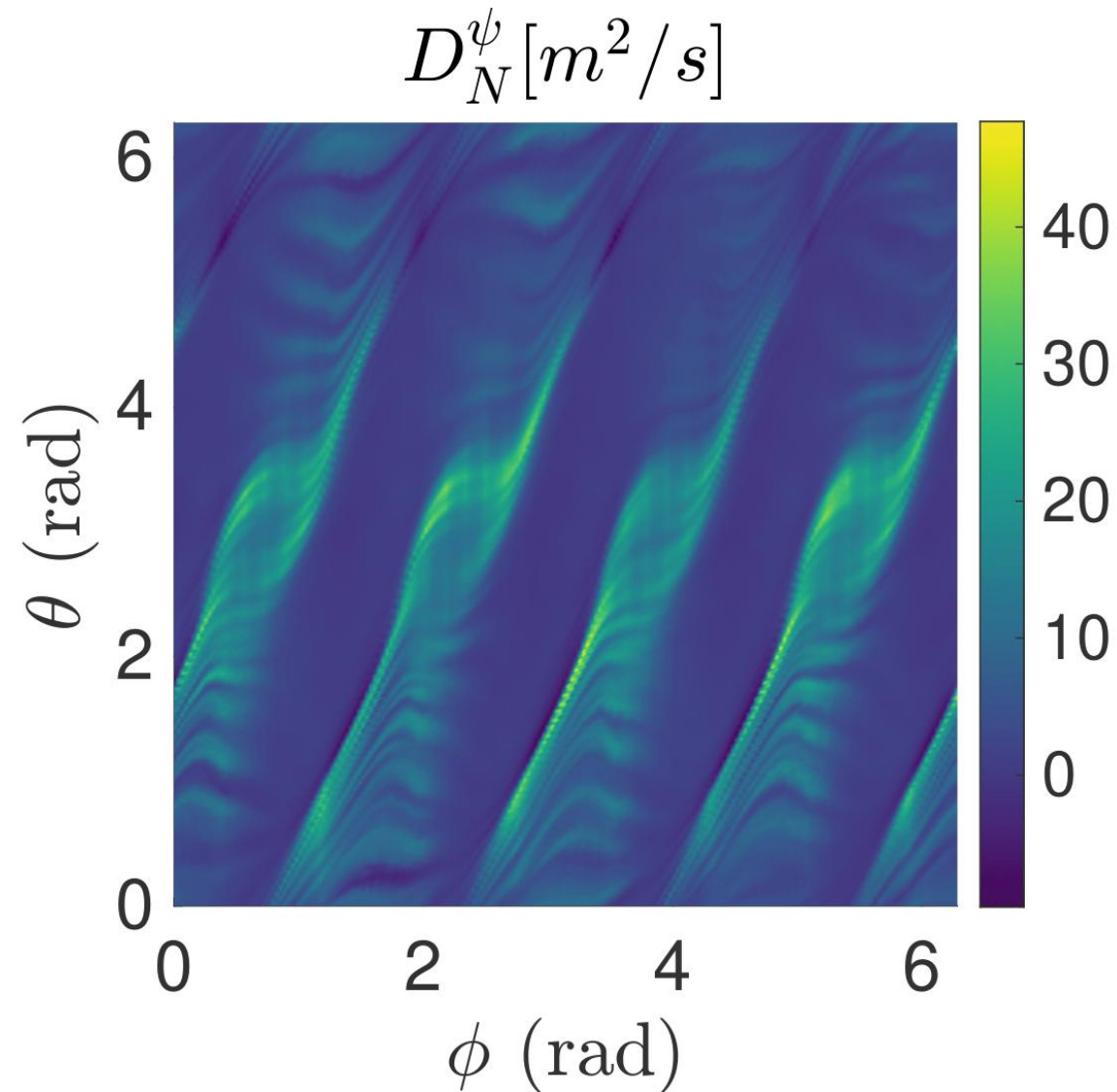




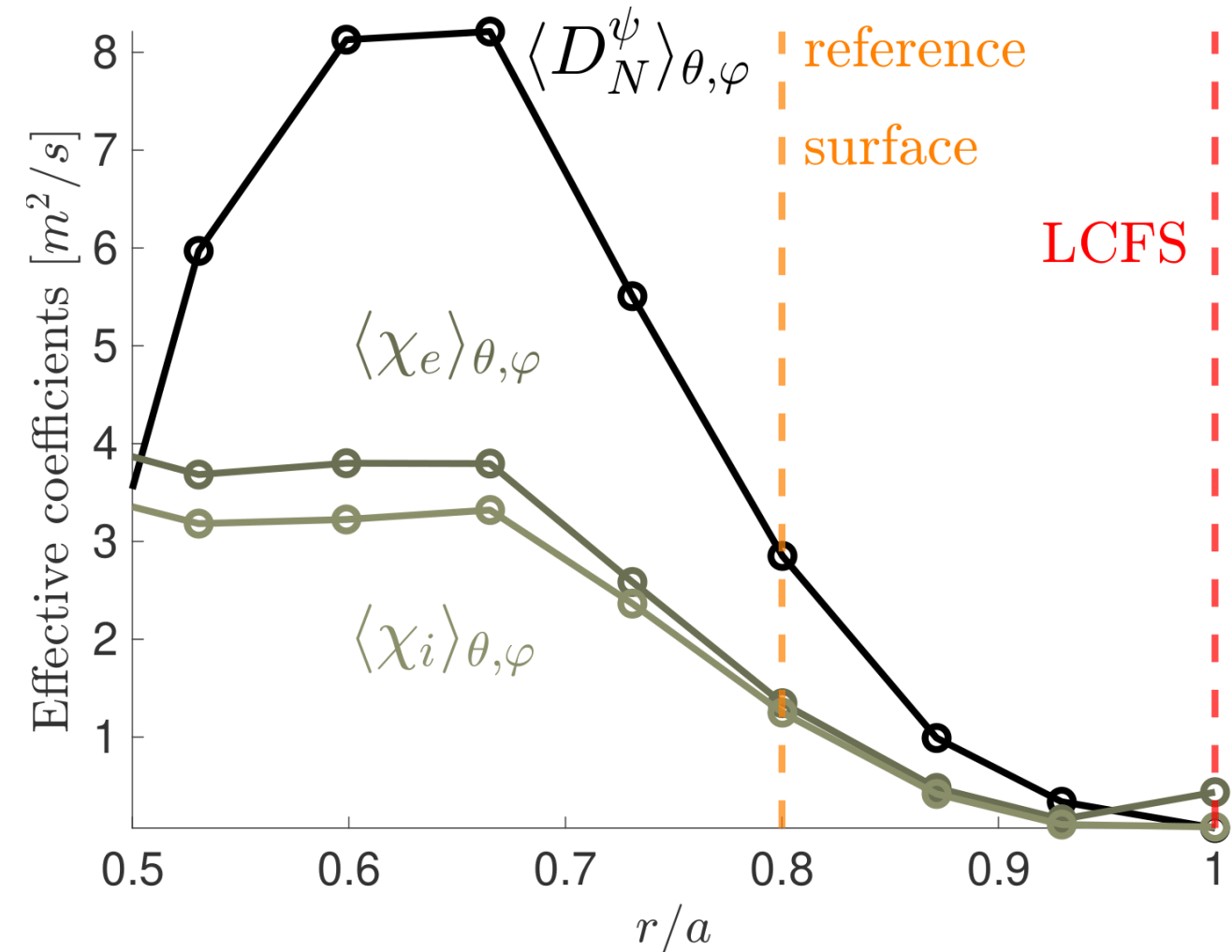
- Negative edge radial electric field



- **More energy** around $f \approx 200 \text{ kHz}$ with cascade of $\sim f^{-4}$
- **Poloidal correlation** $\rho_s \langle k_\theta \rangle \approx 0.12$ and $L_\theta \approx 6 \rho_s$ consistent with measurements of $\rho_s \langle k_\theta \rangle \approx 0.14$ and $L_\theta \approx 7 - 8$



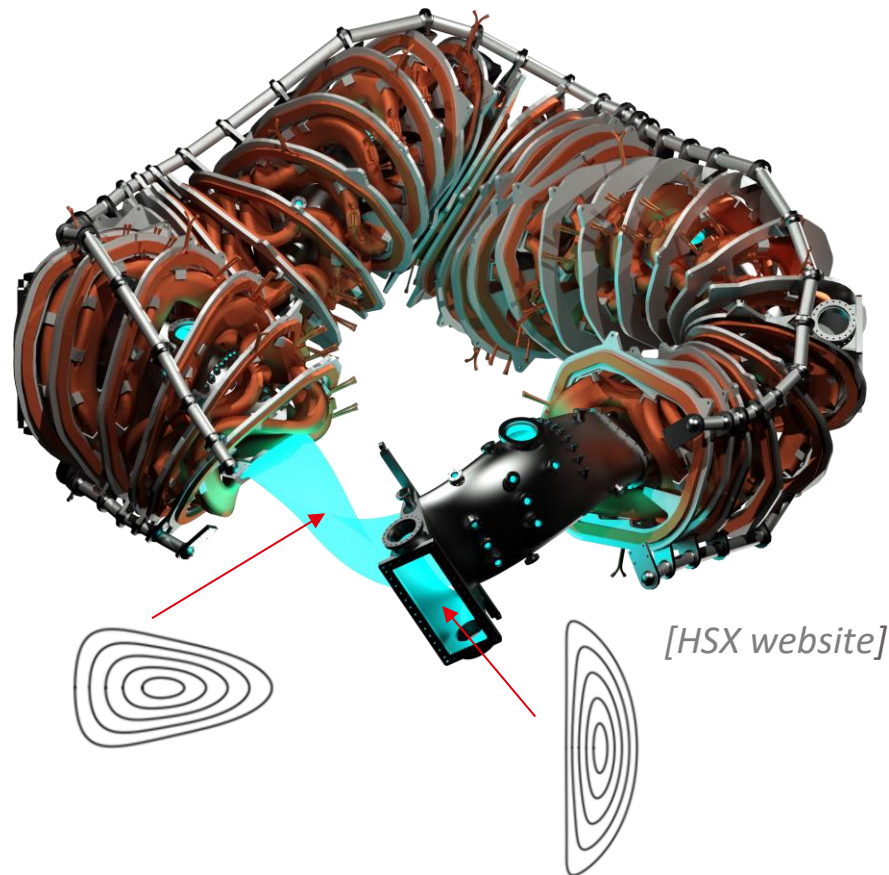
- Regions of negative particle transport



$$D_N^\psi = -\langle \tilde{\Gamma}_N^\psi \rangle_t / \langle \nabla N \cdot \mathbf{n}^\psi \rangle_t$$

$$\chi_{e,i}^\psi = -\langle \tilde{T}_{i,e} \tilde{V}_{E \times B}^\psi / \nabla T_{i,e} \cdot \mathbf{n}^\psi \rangle_t$$

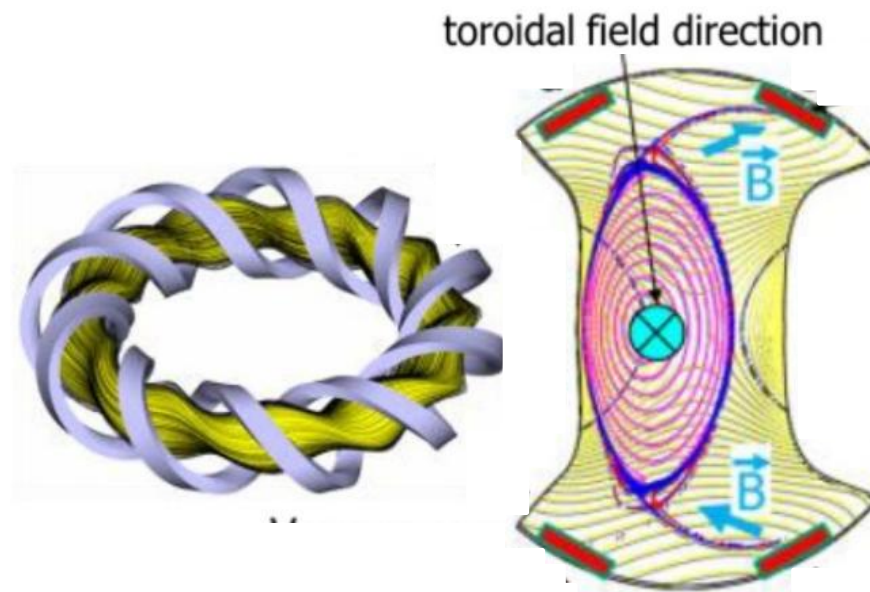
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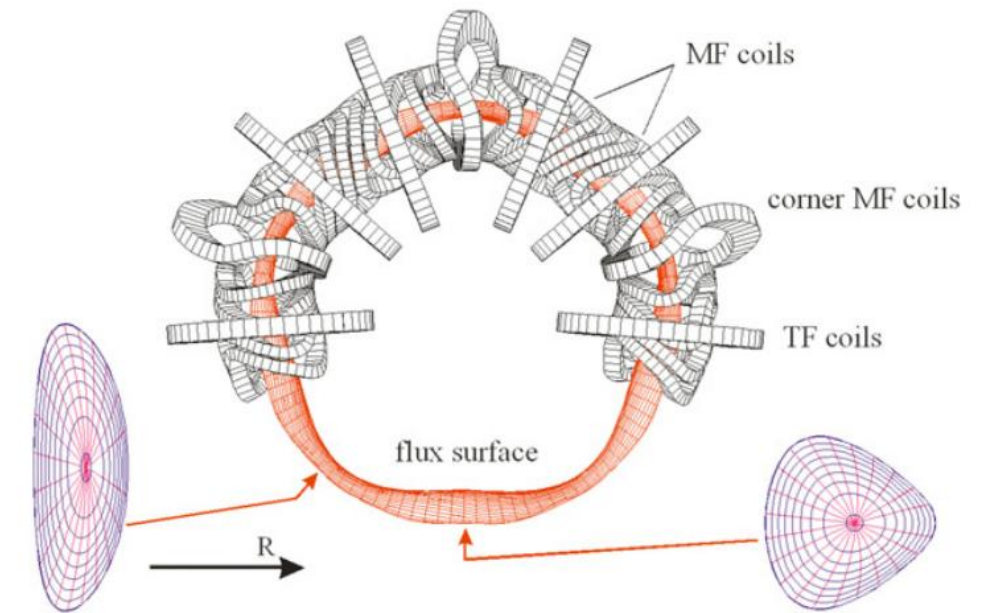
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W7-AS



[Hirsch et al, PPCF 2008]

Predecessor of **W7-X** sharing many features
but smaller in size

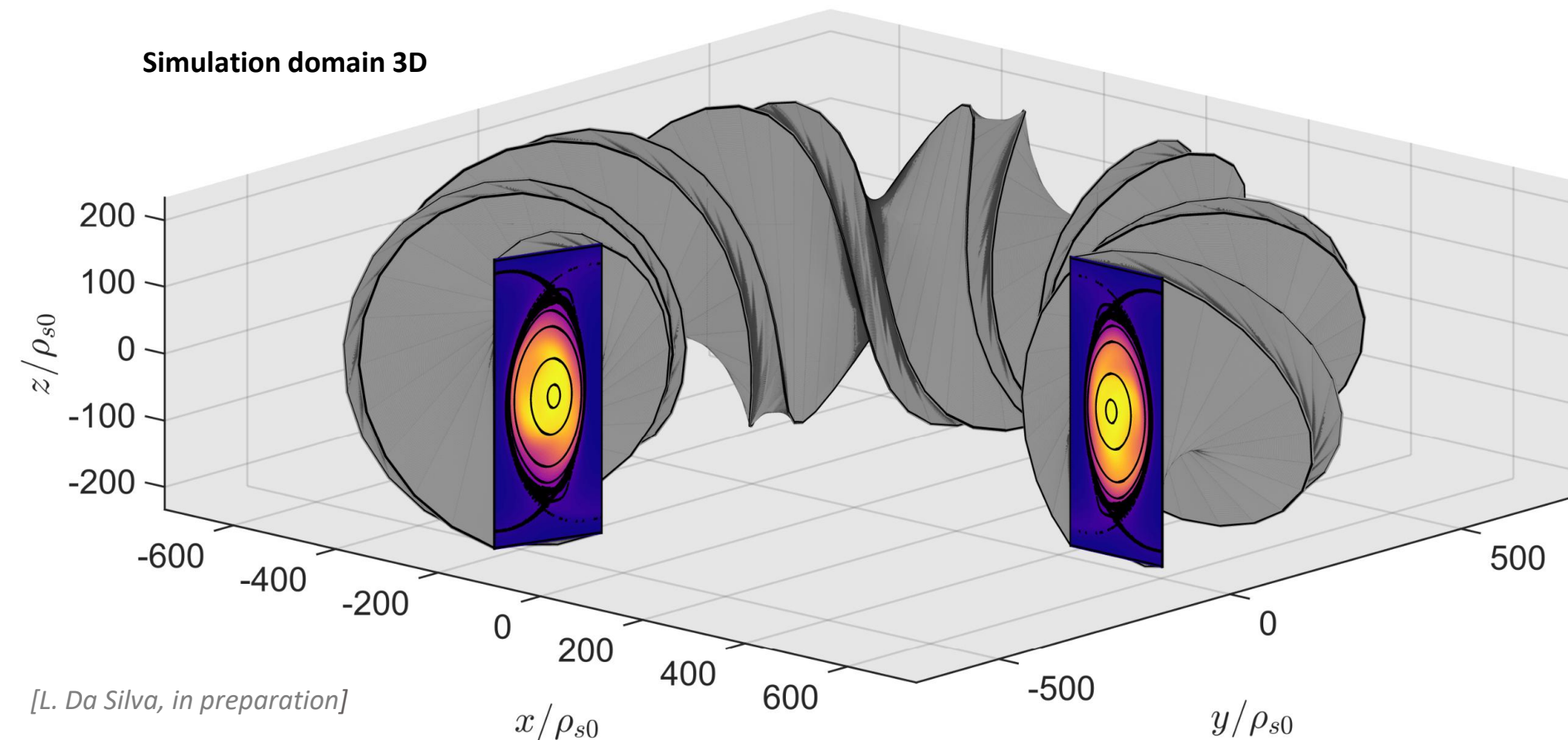
Auxiliary coil currents allowed **great flexibility**
in the edge topology

EPFL Simulating LHD for studying plasma dynamics in the chaotic divertor

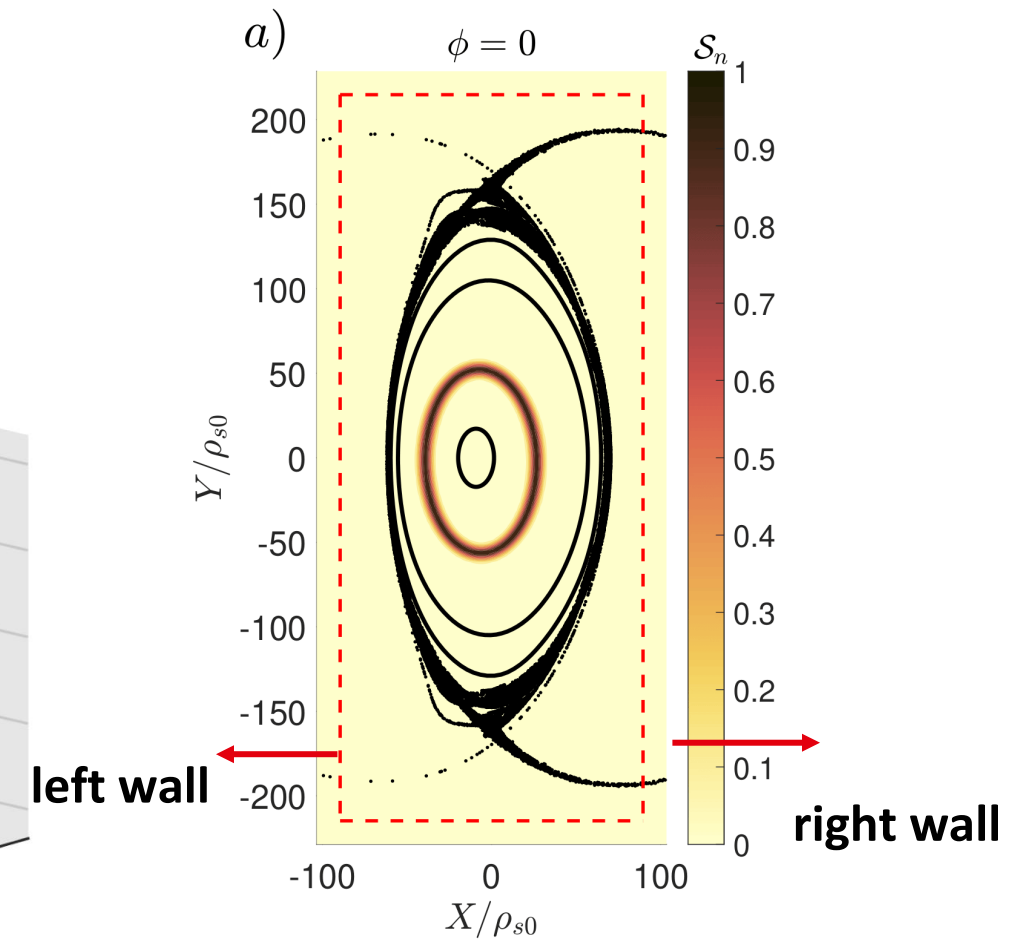
18

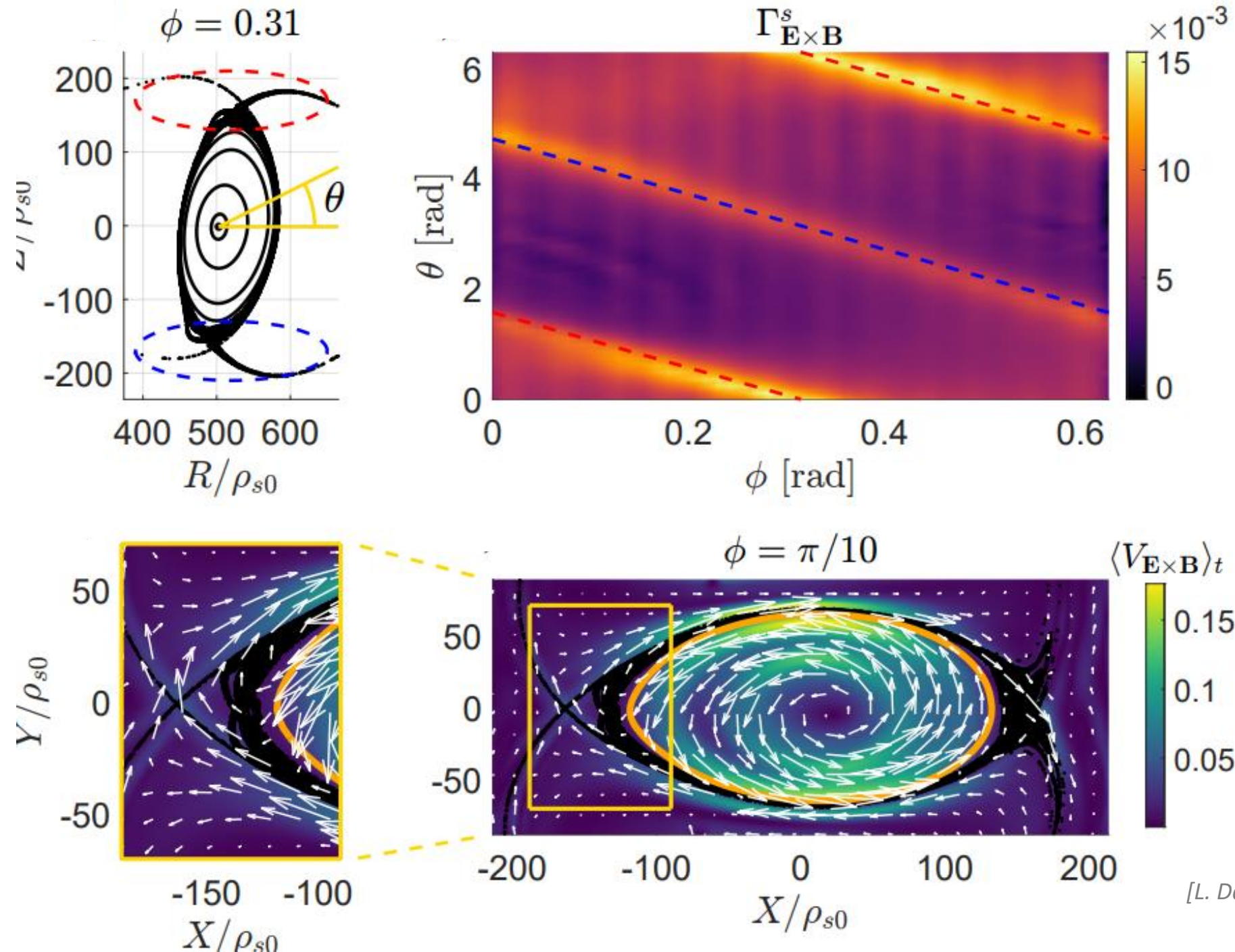
- Simulation domain follows **the divertor plates geometry**
- Considering **1/3 of real size** for gaining computational time
- **Qualitative comparison** with experimental results

Simulation domain 3D



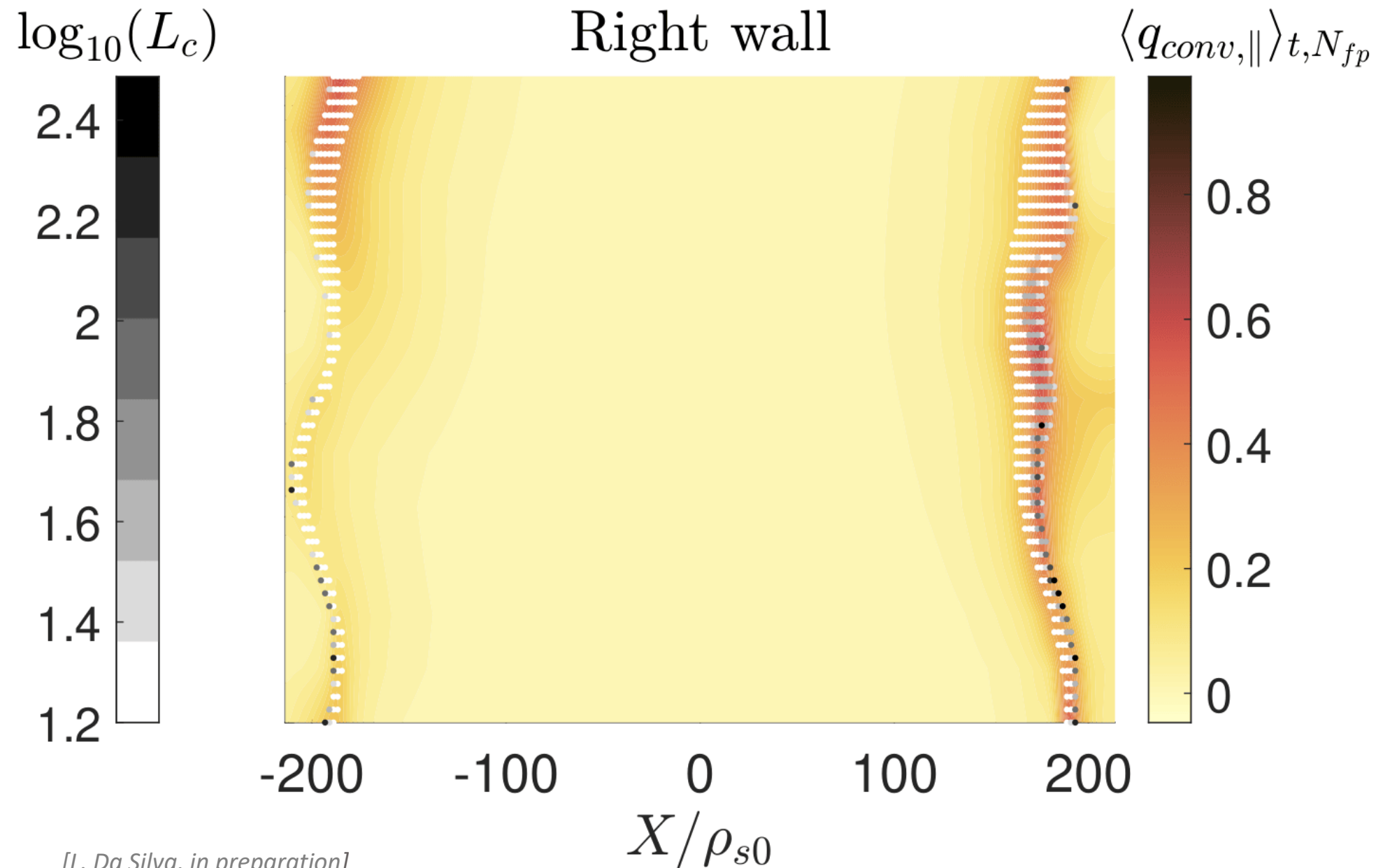
Simulation domain 2D





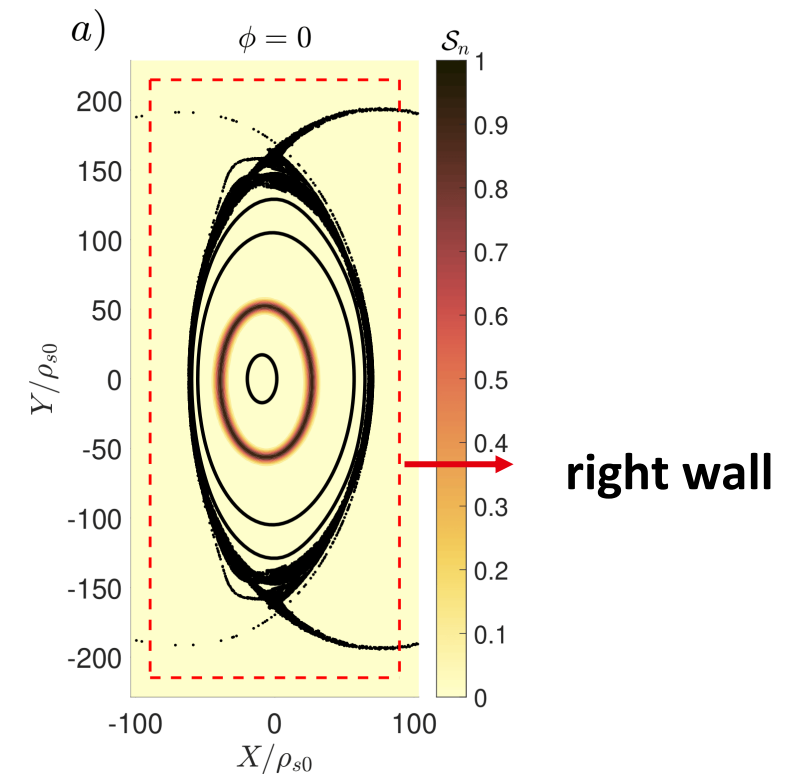
- $\mathbf{E} \times \mathbf{B}$ creating L-R and T-B asymmetries in profiles at the divertor plates
- Consequence of **self-consistently account for drifts**
- **Effect observed experimentally** [S Masuzaki et al. Nuclear Materials and Energy 2019]

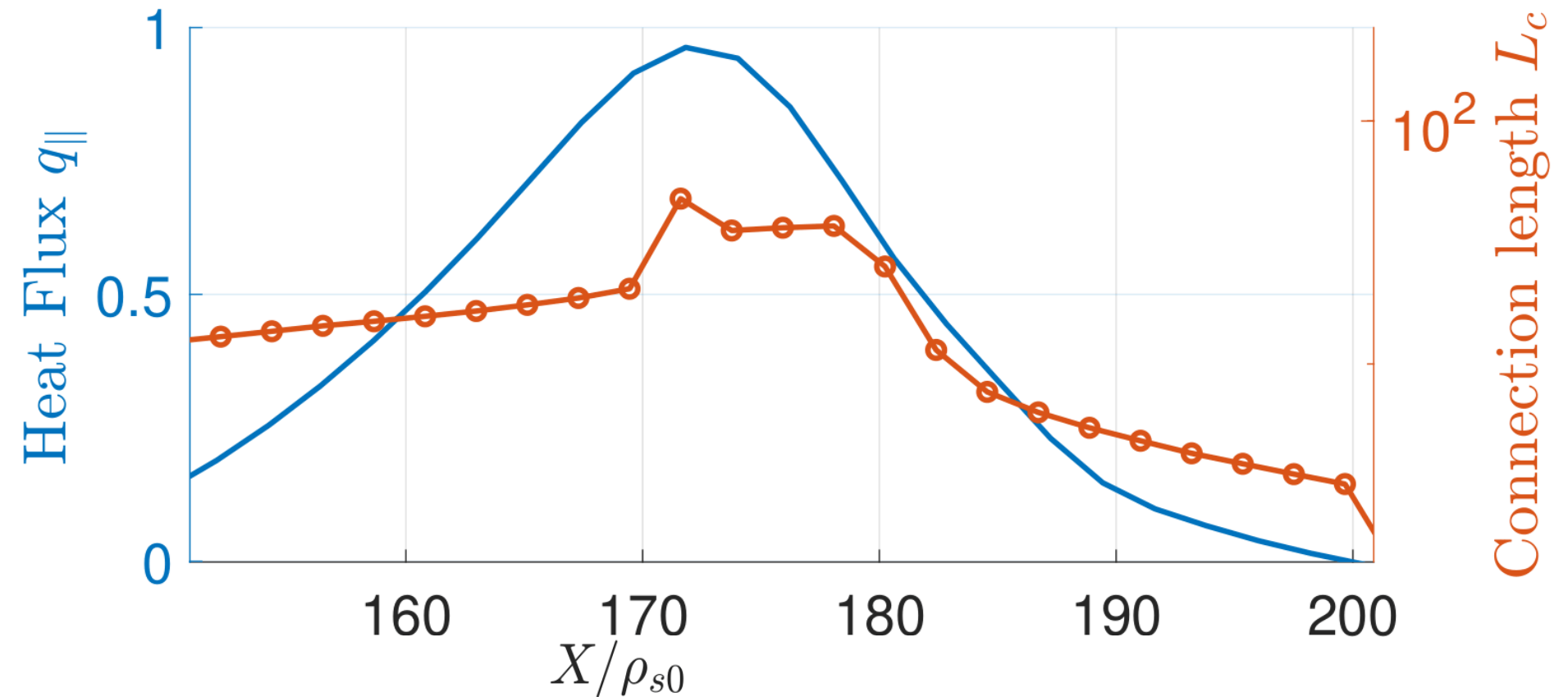
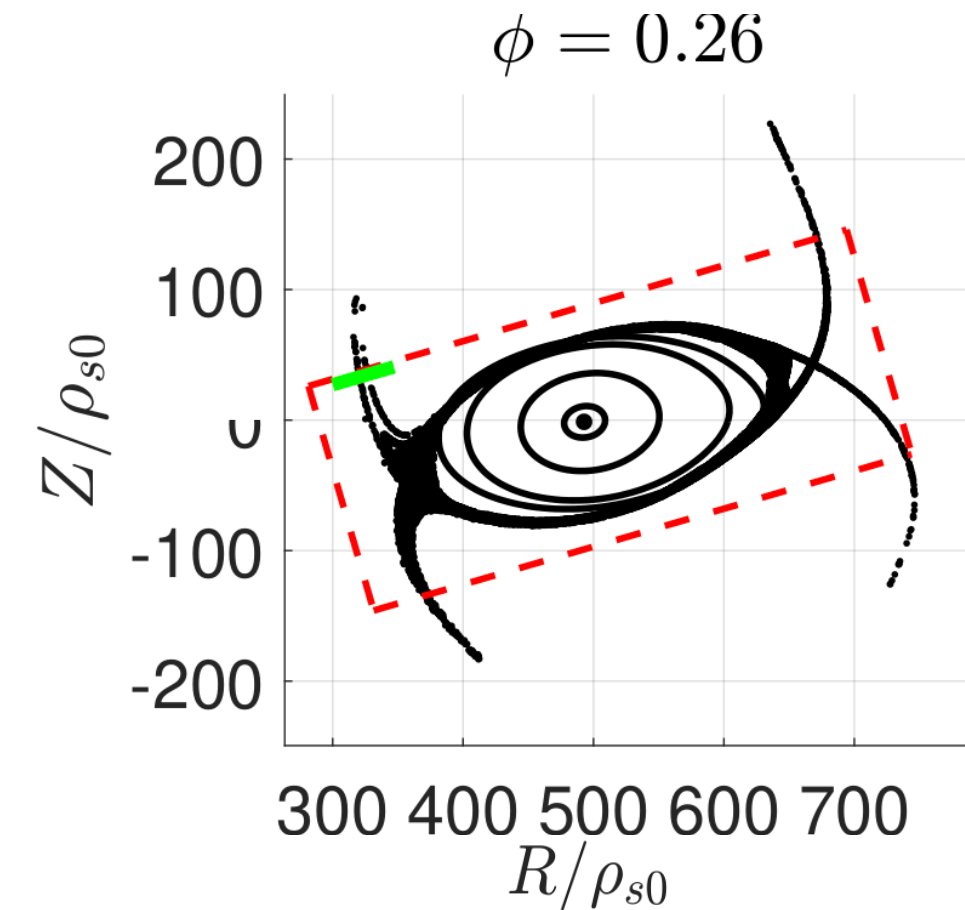
Asymmetries in the heat flux deposition on the divertor plates



[L. Da Silva, in preparation]

- Heat flux deposition correlates with connection length
- Heat profile spreading due to turbulent effects [M. Kobayashi et al. 2022 PRL]

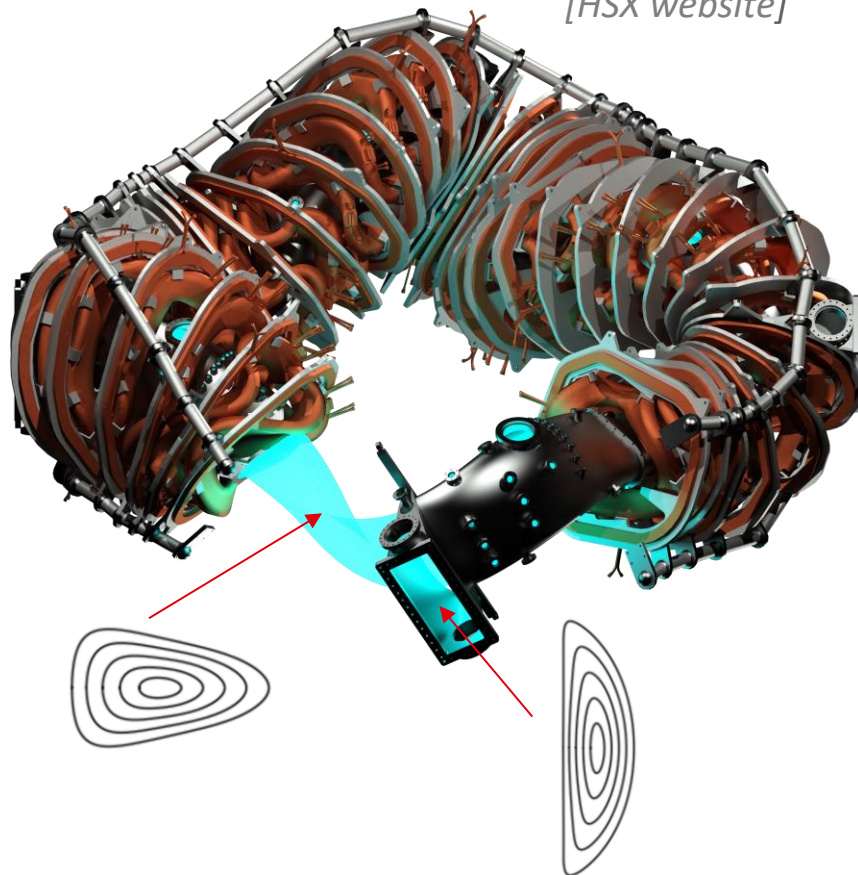




- $\lambda_q = 15\rho_s$ **consistent with experimental results of** $\lambda_q \approx 10 - 50\rho_s$ [M. Kobayashi et al. 2022 PRL]

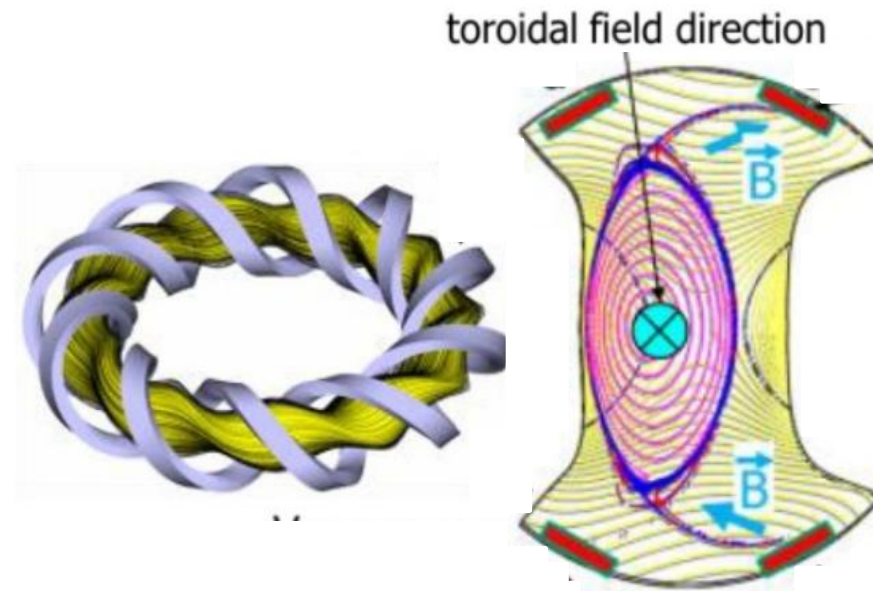
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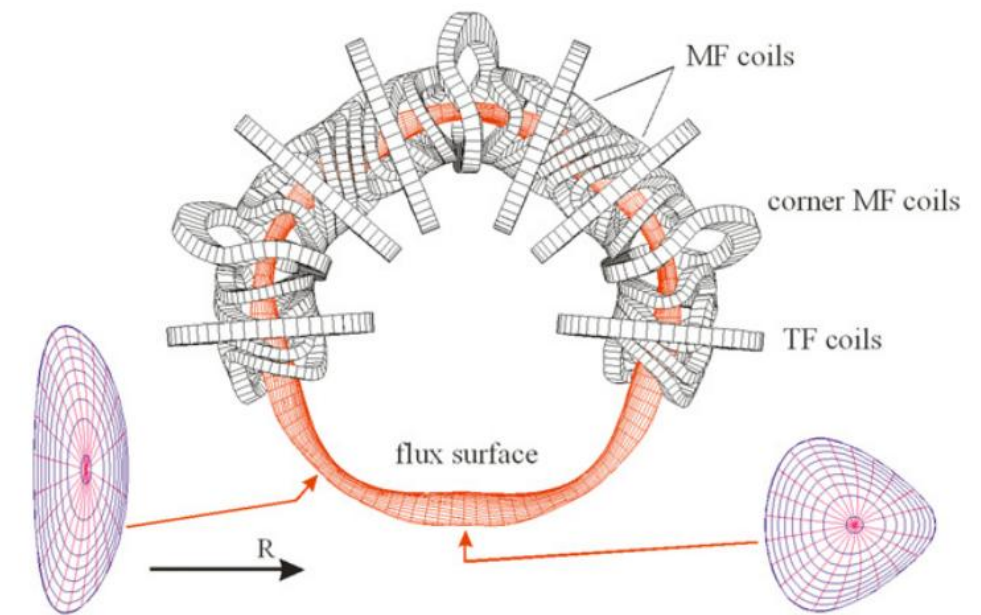
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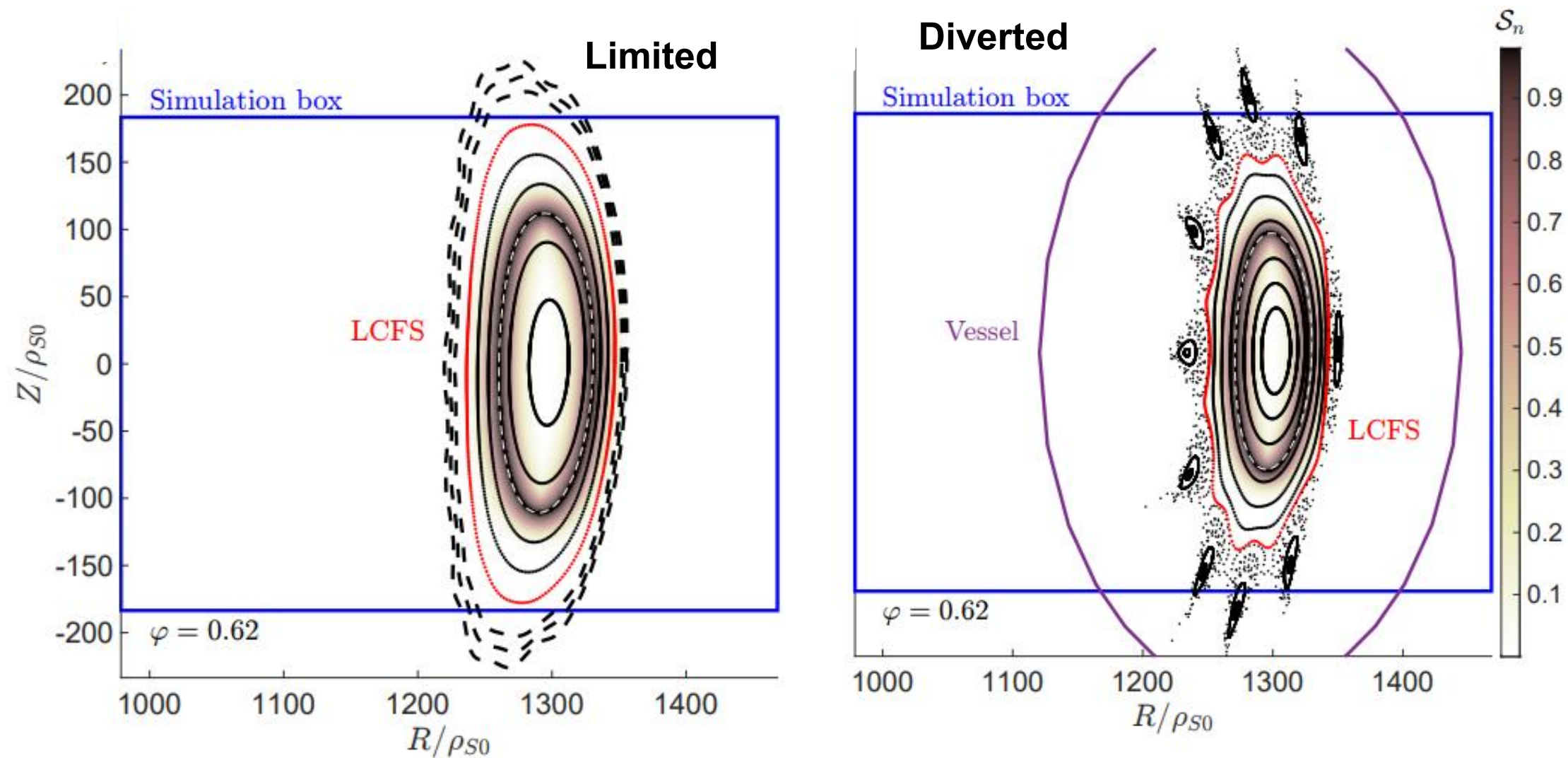
W7-AS



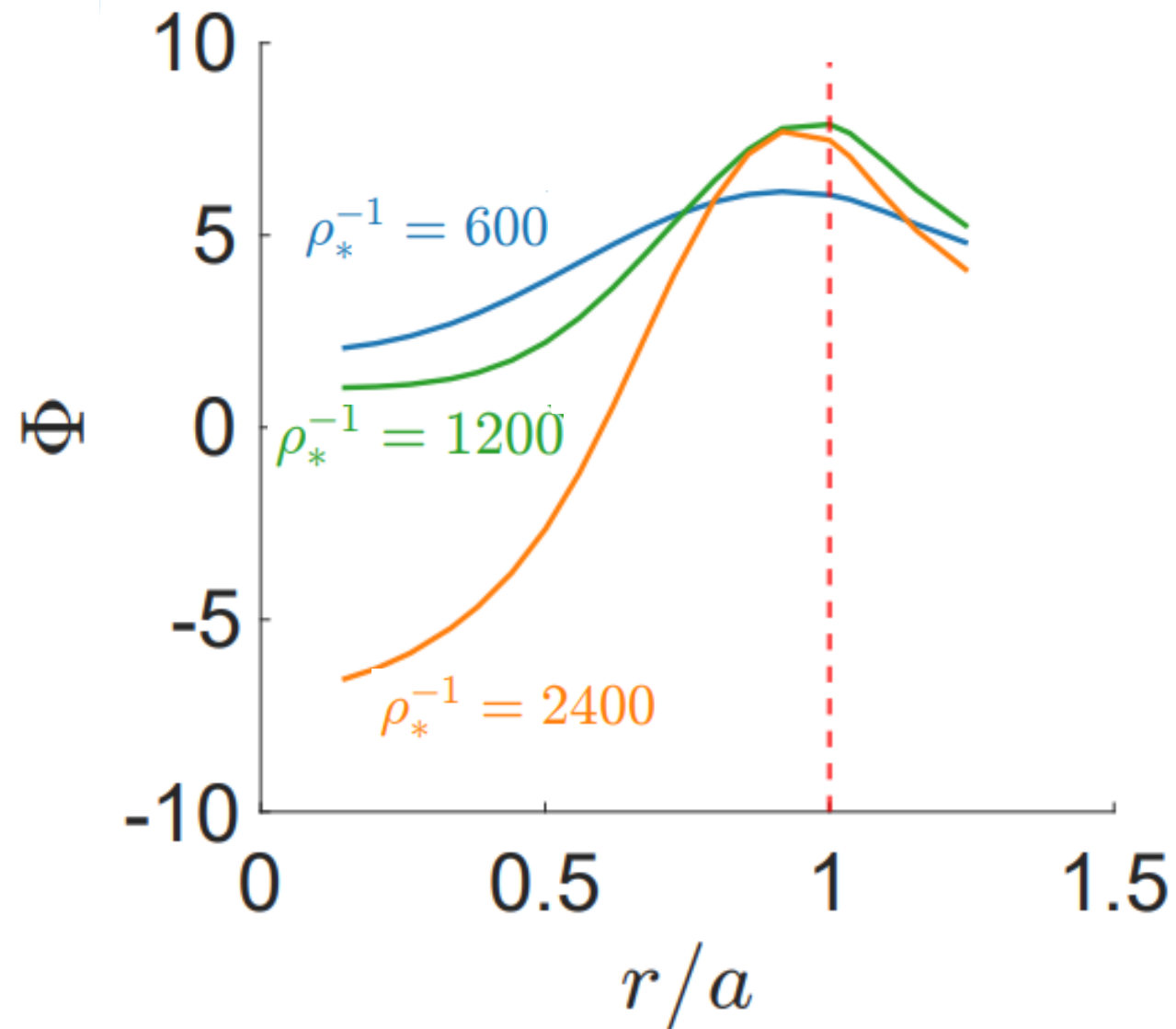
[Hirsch et al, PPCF 2008]

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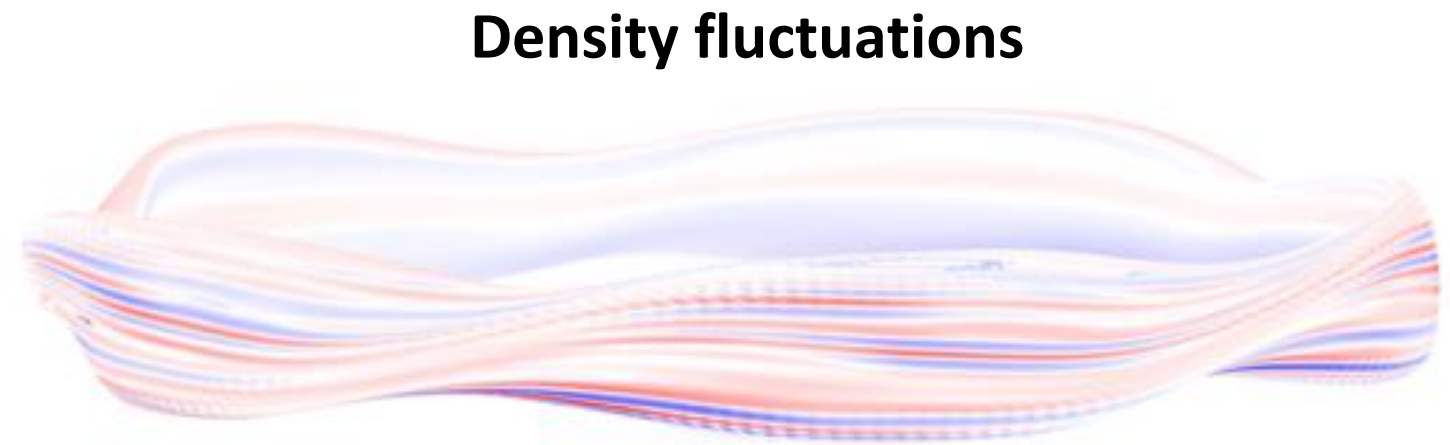
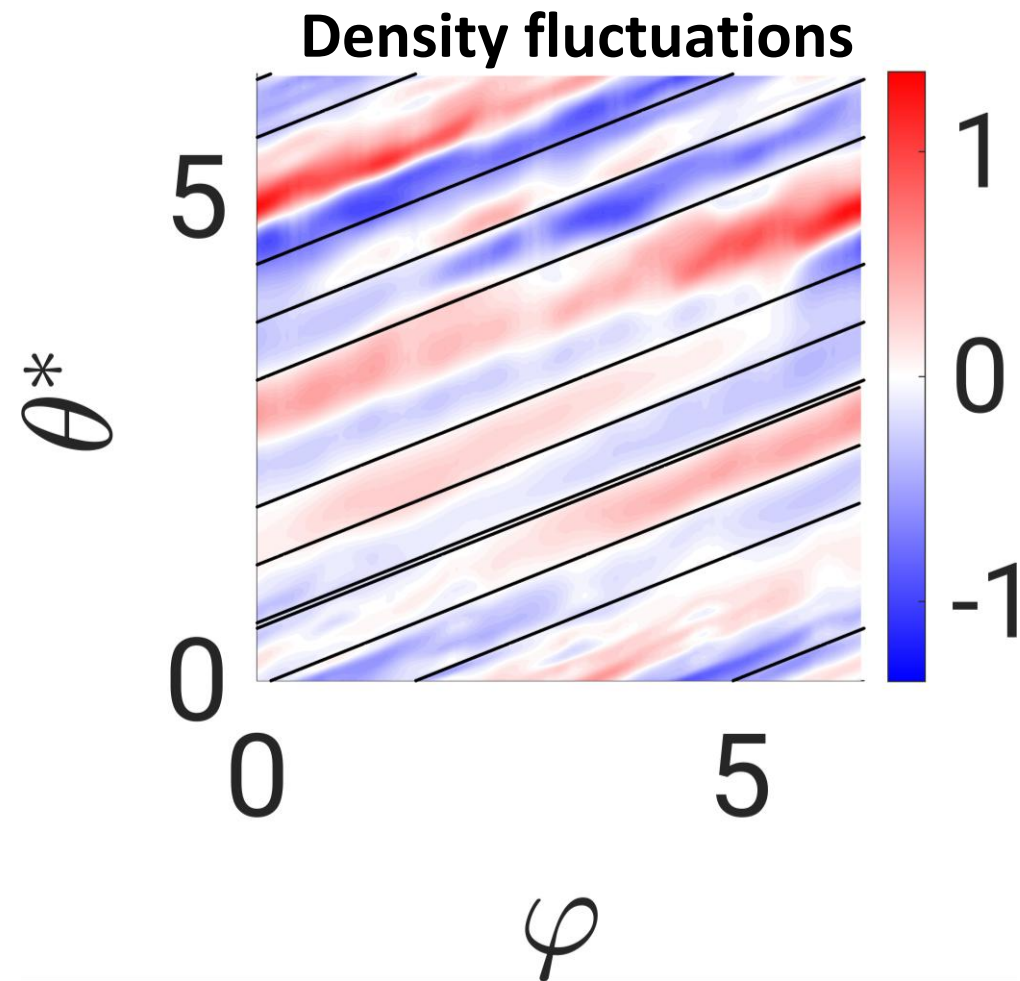
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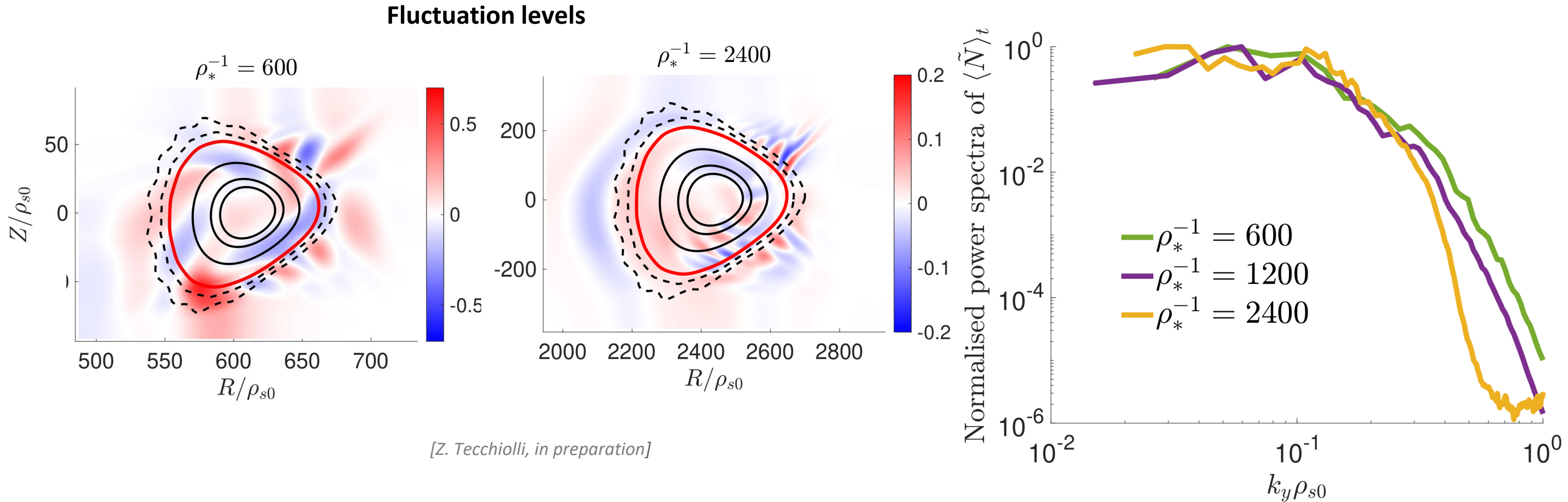
- **Limited configuration:** quarter-size $R_0/\rho_{s0} \sim 600$, half-size $R_0/\rho_s = 1200$, and full-size $R_0/\rho_{s0} \sim 2400$ with $\iota \sim 0.4$
- **Island-diverted configuration:** $R_0/\rho_{s0} \sim 2400$ and $\iota \sim 0.5$



- **Negative radial electric field in the core** following $E_r \sim \partial_r P_i / N$ and **positive in the SOL** following $\Phi \sim \lambda T_e$, forming **layer shearing fluctuations** [J Bleuel et al 2002 New J. Phys.]
- $E \times B$ **main radial transport** mechanism [M. Schubert et al. 2006]

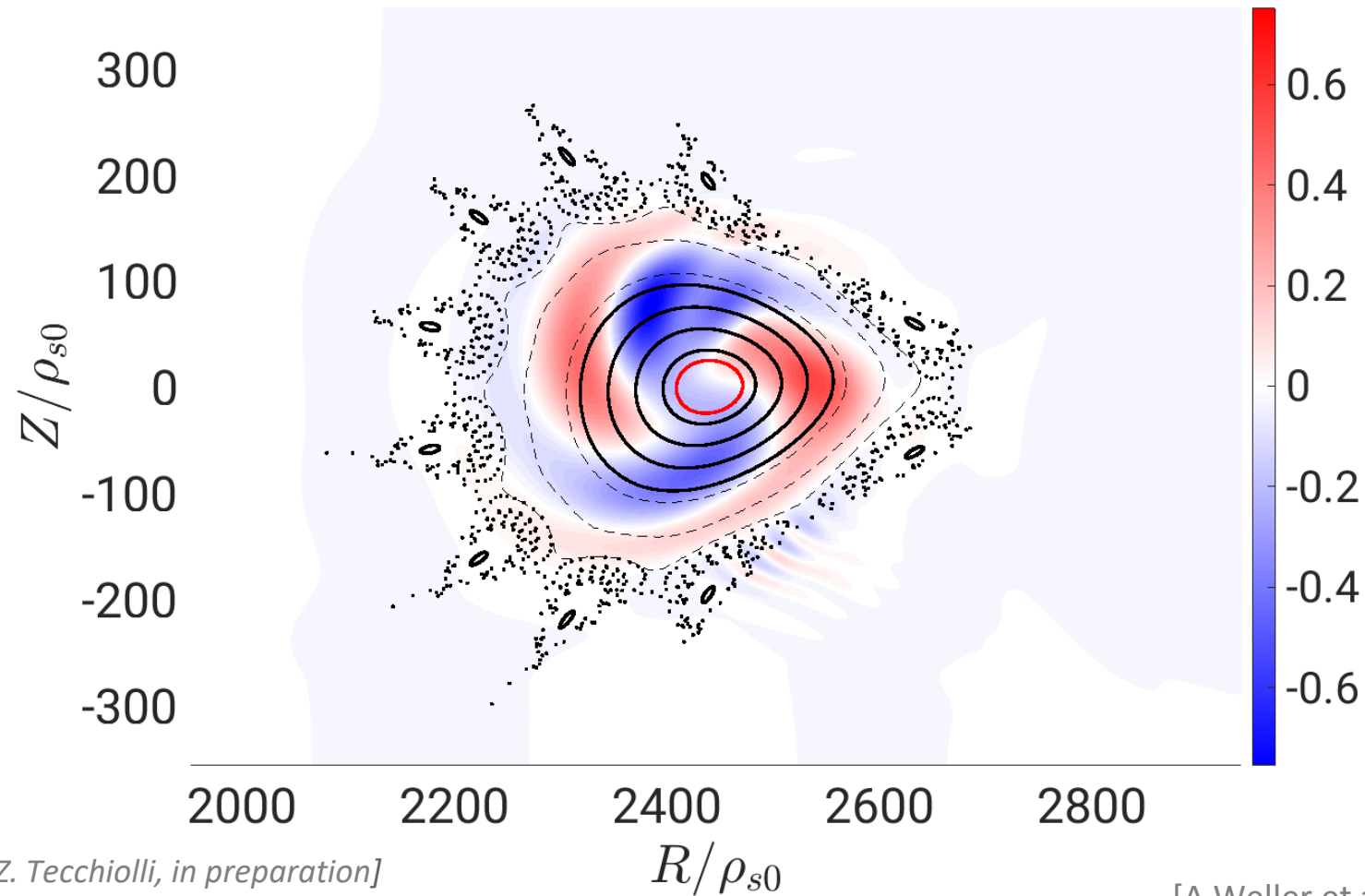


- **Field-aligned, $k_{\parallel} \sim 0$, curvature driven, $(\mathbf{b} \cdot \nabla \mathbf{b}) \cdot \nabla P$ turbulence**
- **Dominant mode number following $\iota \sim n/m$ where n breaks the field periodicity of 5**
- **Importance of full torus simulations**

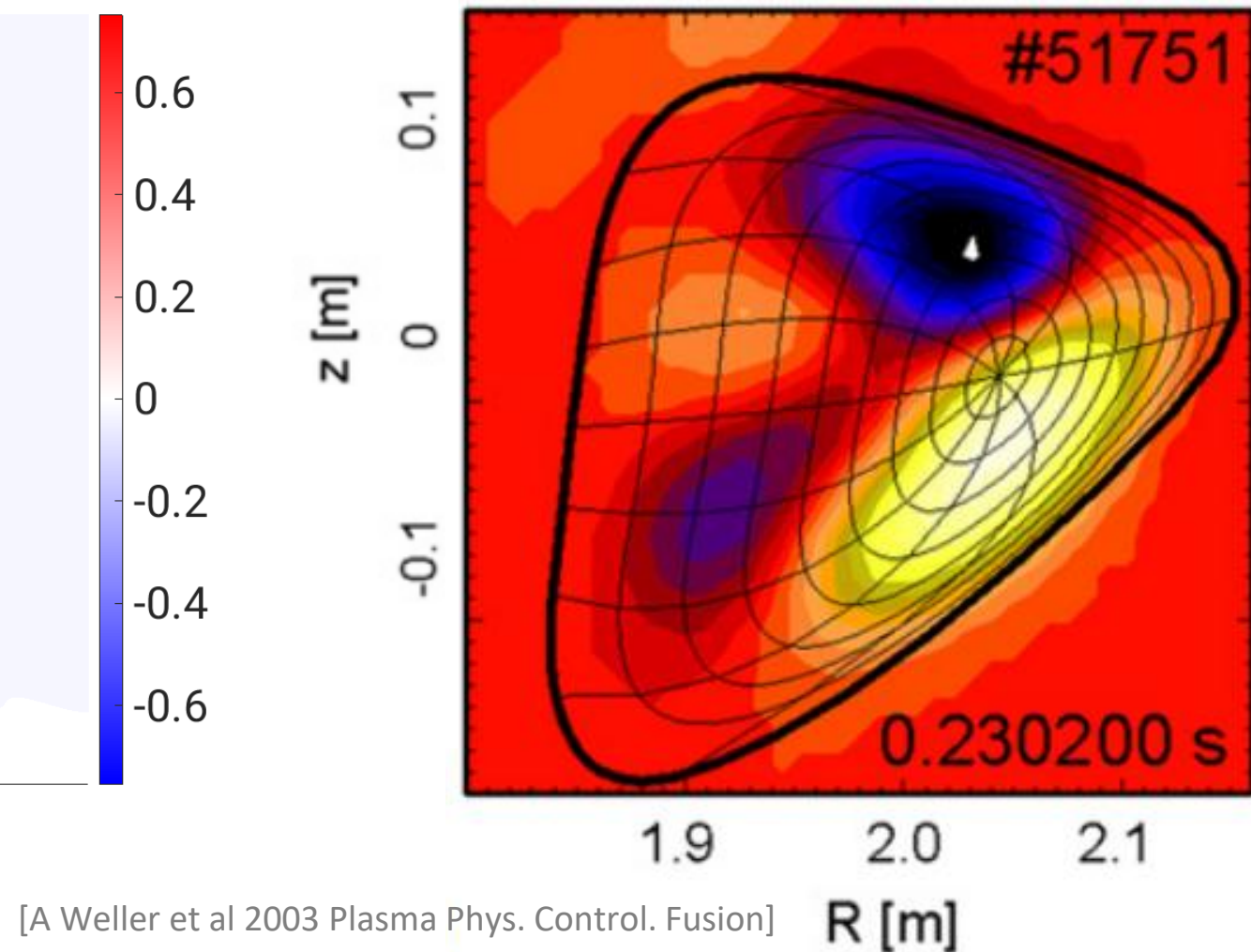


- Poloidal size of fluctuating structure similar in ρ_s units among the different sizes
- **Broader turbulent spectrum** compared to previous simulations and **consistent with experiments** [J Bleuel, et al. New Journal of Physics (2002)]

Total density fluctuations



x-ray tomography



- **Global** $(m, n) = (2, 1)$ **field-aligned instability** consistent with $\iota \sim 0.5$ and unstable Mercier condition for low-beta plasma in W7-AS
- **Study of SOL region still undergoing**

Conclusion: successful edge validation in W7-AS, LHD, and HSX provides a strong basis for further physical understanding

- Qualitative **validation** in **W7-AS** and **LHD**, combined with quantitative validation in **HSX**, indicates that the **drift-reduced Braginskii model** represents a valid description for plasma physics in the edge of stellarators.
- **Curvature-driven and field-aligned instabilities constitute the primary energy source** in the simulations considered. However, **plasma parameters** and **global electromagnetic effects** may significantly influence the resulting dynamics.
- **Substantial physics in the edge region remains to be explored.** Key areas include **neutral dynamics**, detailed characterization of **instabilities**, **saturation mechanisms**, **stochastic magnetic fields**, and **divertor geometries**.