

# Investigations of plasma turbulence in the edge of HSX, LHD and W7-AS

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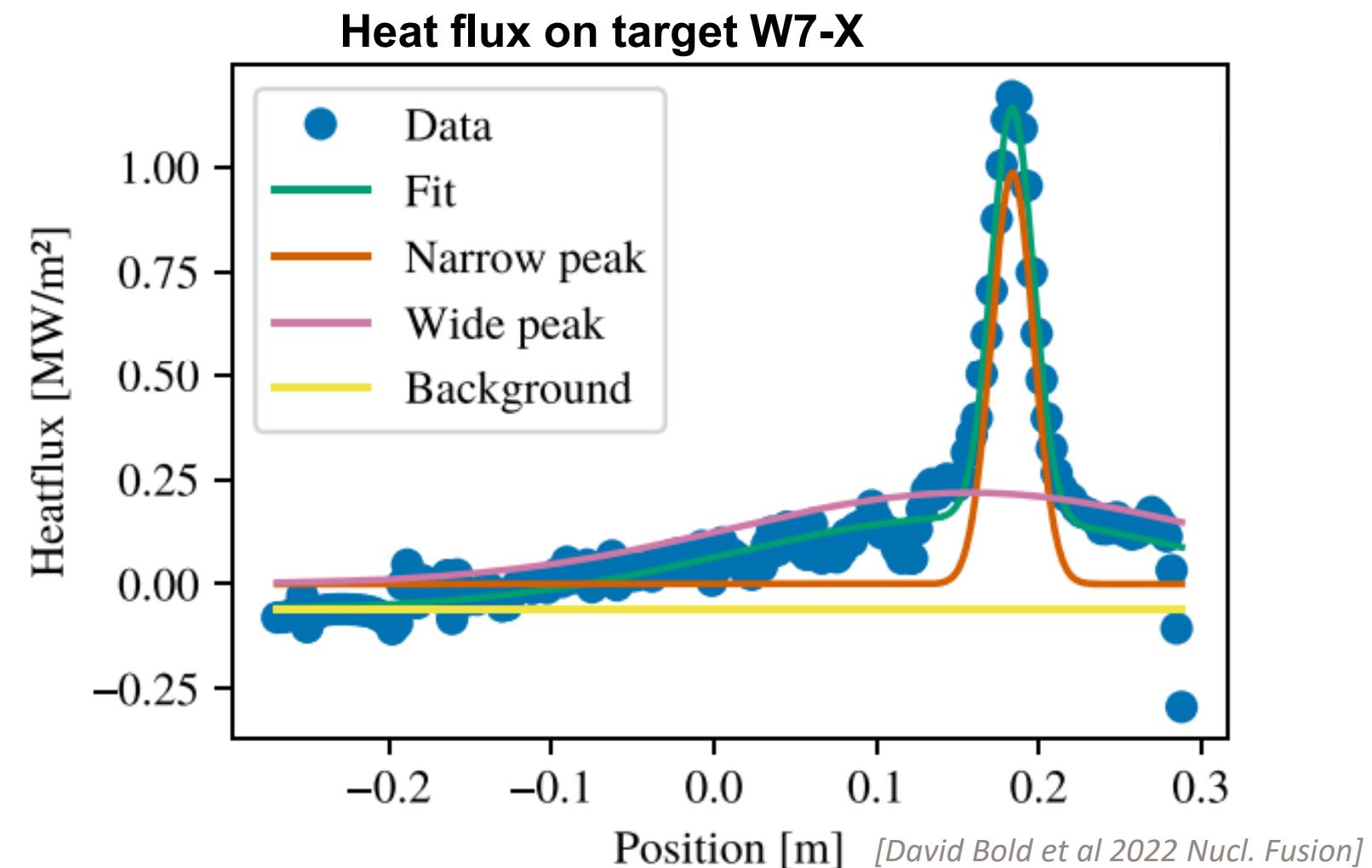
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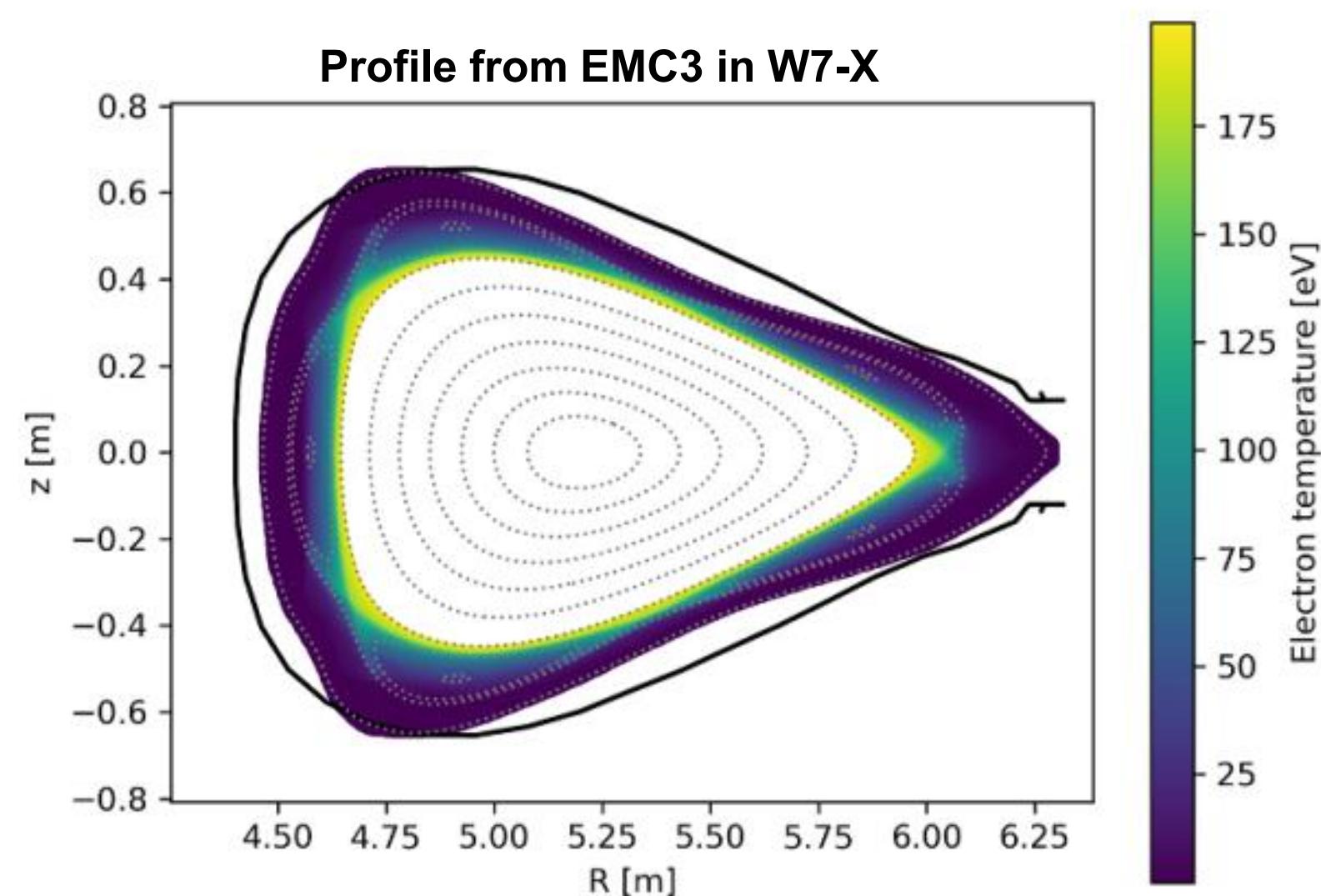
<sup>5</sup>Gauss Fusion GmbH, Garching bei München, Germany,

- **Significant heat fluxes in the edge of stellarators**
- Perpendicular transport influences **peaks and spread of heat flux profile and SOL width**
- The **core plasma** is constrained by **boundary conditions established through edge turbulence**



- **Inclusion of turbulence positively affecting the divertor design**
- **Include edge turbulence metrics in the stellarator optimisation loop** [M. J. Gerard et al, Nucl. Fusion 2023]

- Recent and past investigations are mainly based on **transport codes** [Feng, Y., et al. *Contributions to Plasma Physics* (2004)]
- Turbulent transport is modelled with **ad-hoc turbulent coefficients**
- Use turbulent codes to inform transport codes.**

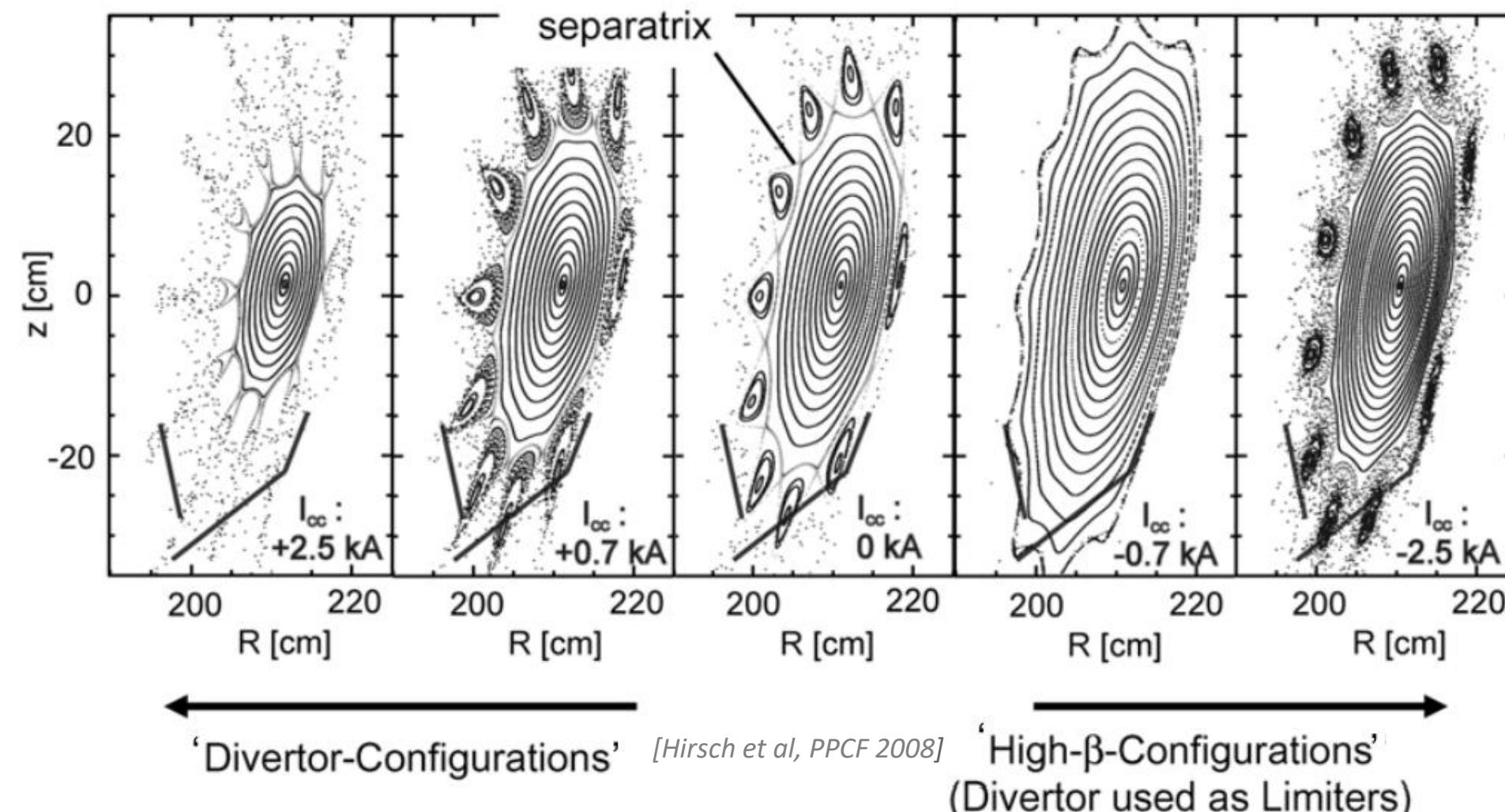


What makes the edge region challenging?

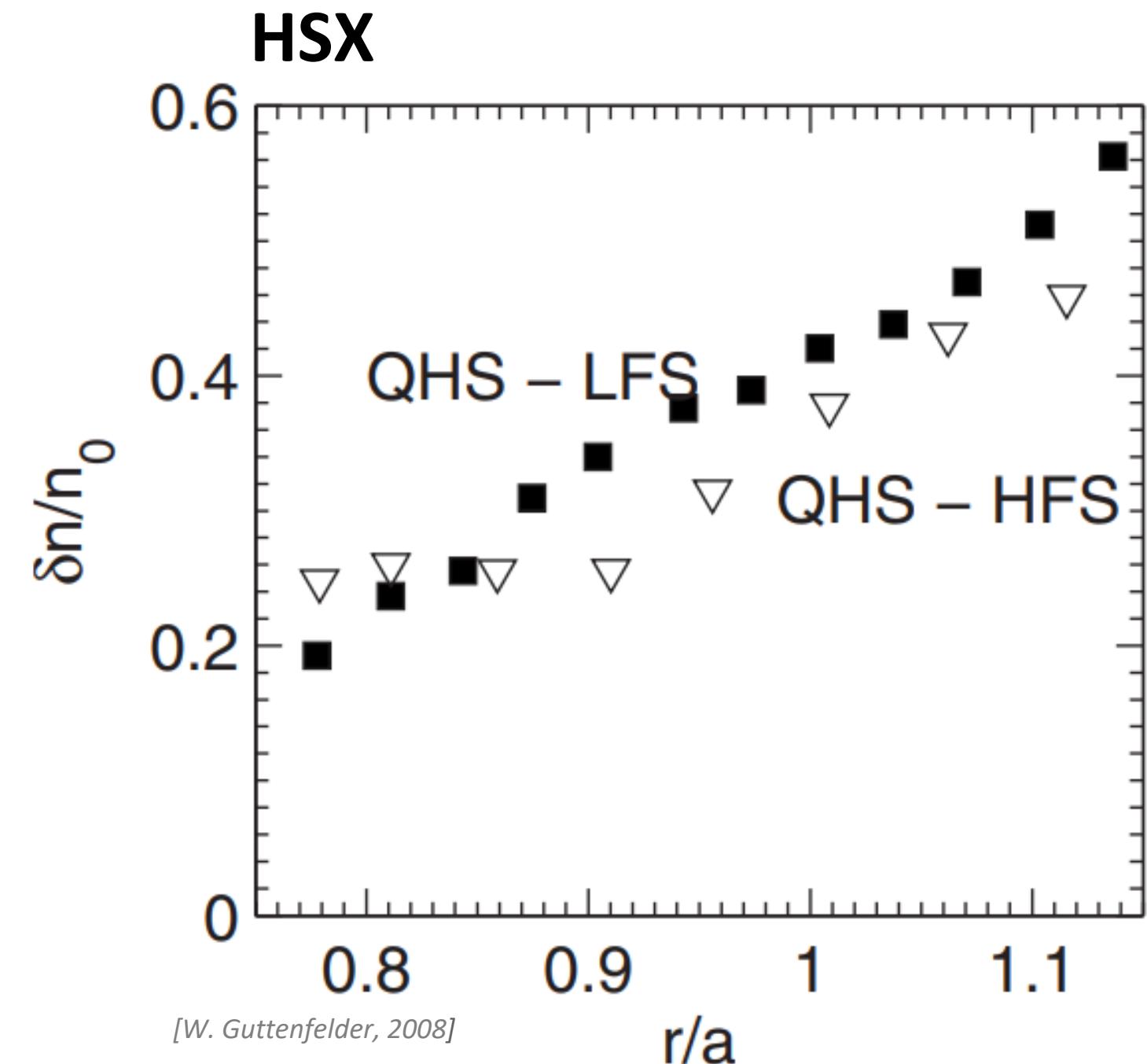
[David Bold et al 2024 Nucl. Fusion]

- Edge complex magnetic structures: magnetic islands, stochastic regions, X-points

W7-AS

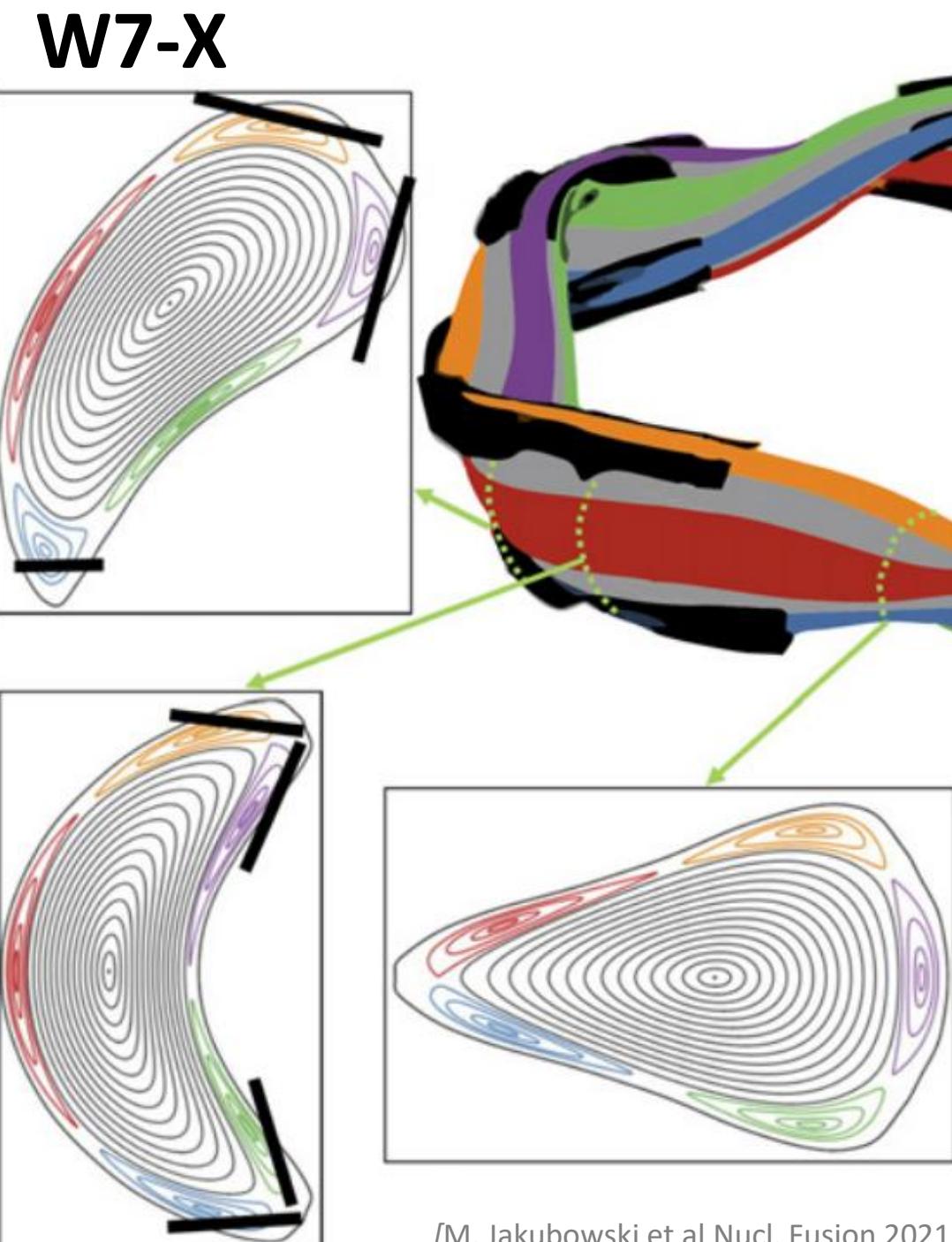


- **Edge complex magnetic structures:** magnetic islands, stochastic regions, X-points
- **Large fluctuations levels:** no separation between background and fluctuations

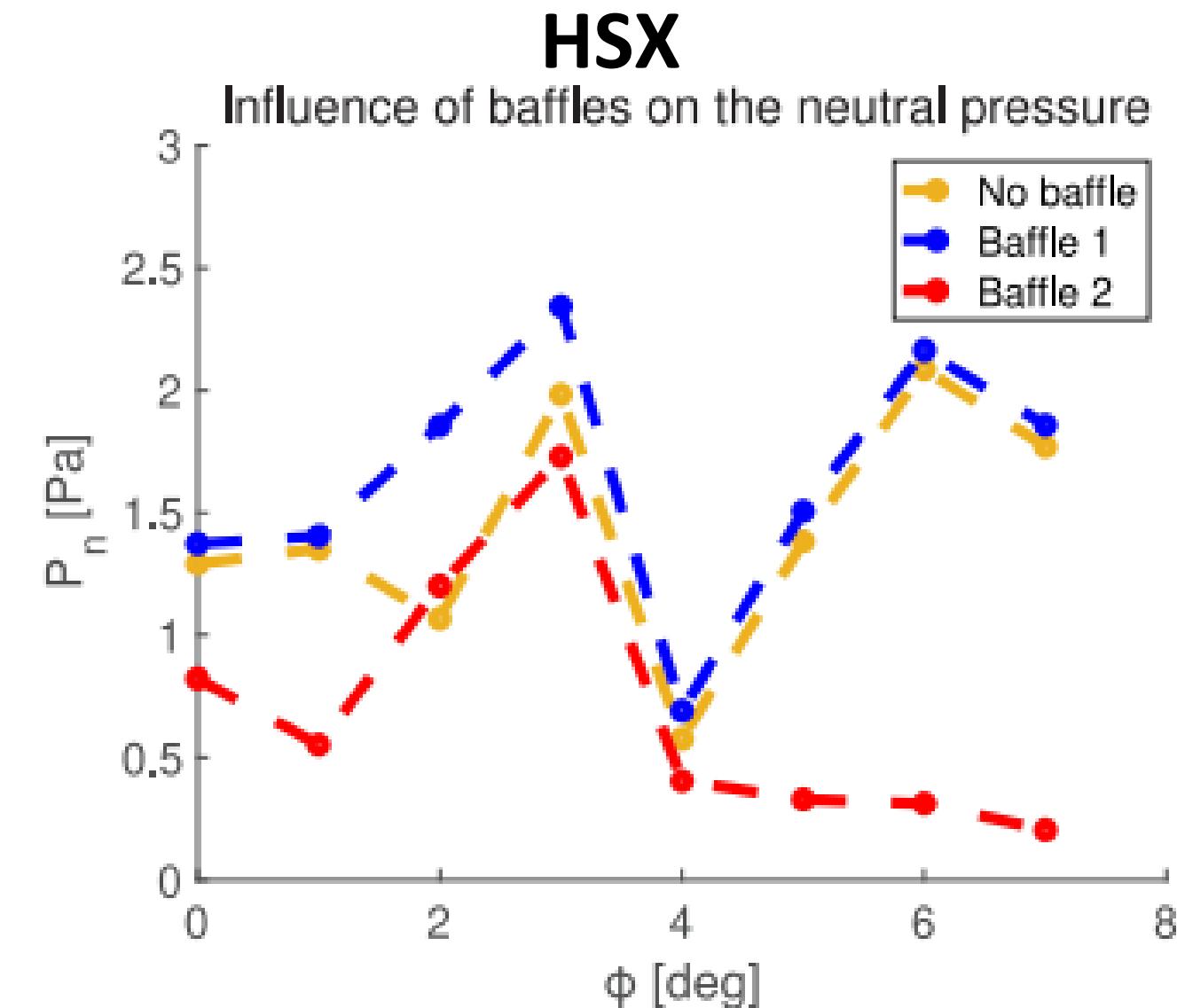


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## Simulation requirements

- **Flexible discretization scheme with non-flux-aligned coordinates**
- **No separation of equilibrium and fluctuating quantities**
- **Flux-driven:** sources generate gradients for turbulence driving
- **Global:** simulate the entire volume
- **Magnetic pre-sheath boundary conditions**
- **Coupling plasma and neutrals**

# Drift-reduced Braginskii equations to simulate the boundary plasma turbulence

- Plasma edge is **highly collisional**  $\rho_{L,i} k_{\perp} \ll 1$  ,  $\lambda_{mfp} \ll L_{\parallel}$  use **two-fluid Braginskii model**

[S. I. Braginskii. *Reviews of Plasma Physics*, 1965]

- Turbulence time scale longer than  $\Omega_{ci}^{-1}$ , **drift-reduced approximation** [A. Zeiler, (1999)]

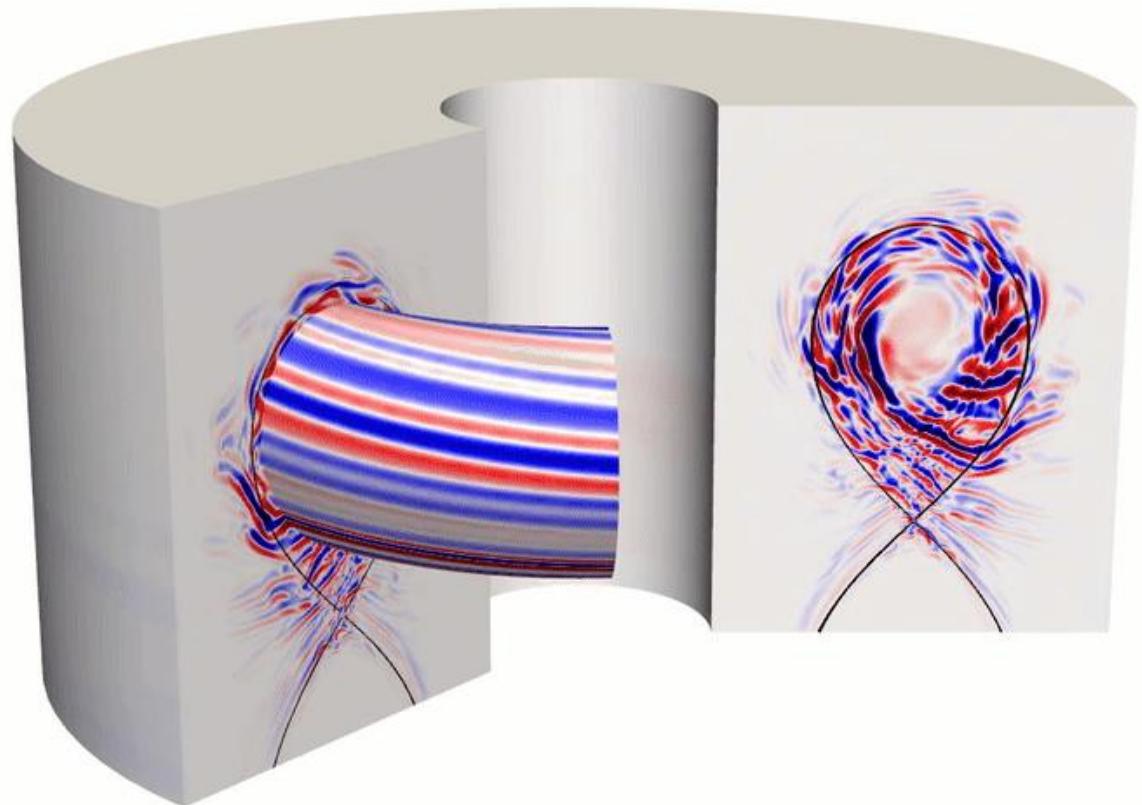
$$\mathbf{V}_e = \mathbf{V}_{E \times B} + \mathbf{V}_{de} + V_{\parallel e} \mathbf{b}, \quad \mathbf{V}_i = \mathbf{V}_{E \times B} + \mathbf{V}_{di} + \mathbf{V}_{pol} + V_{\parallel i} \mathbf{b}$$

- Quasi-neutrality:**  $n_e \sim n_i \sim n$  ,  $\frac{\partial n}{\partial t} + \underbrace{\nabla \cdot \Gamma_{E \times B}}_{\text{E} \times \text{B} \text{ convection}} + \underbrace{\nabla \cdot \Gamma_{di}}_{\text{diamagnetic convection}} + \underbrace{\nabla \cdot \Gamma_{pol}}_{\text{polarization convection}} + \underbrace{\nabla_{\parallel} \Gamma_{\parallel}}_{\text{parallel flow}} = \mathcal{S}_n$  .  $\underbrace{\quad}_{\text{source}}$

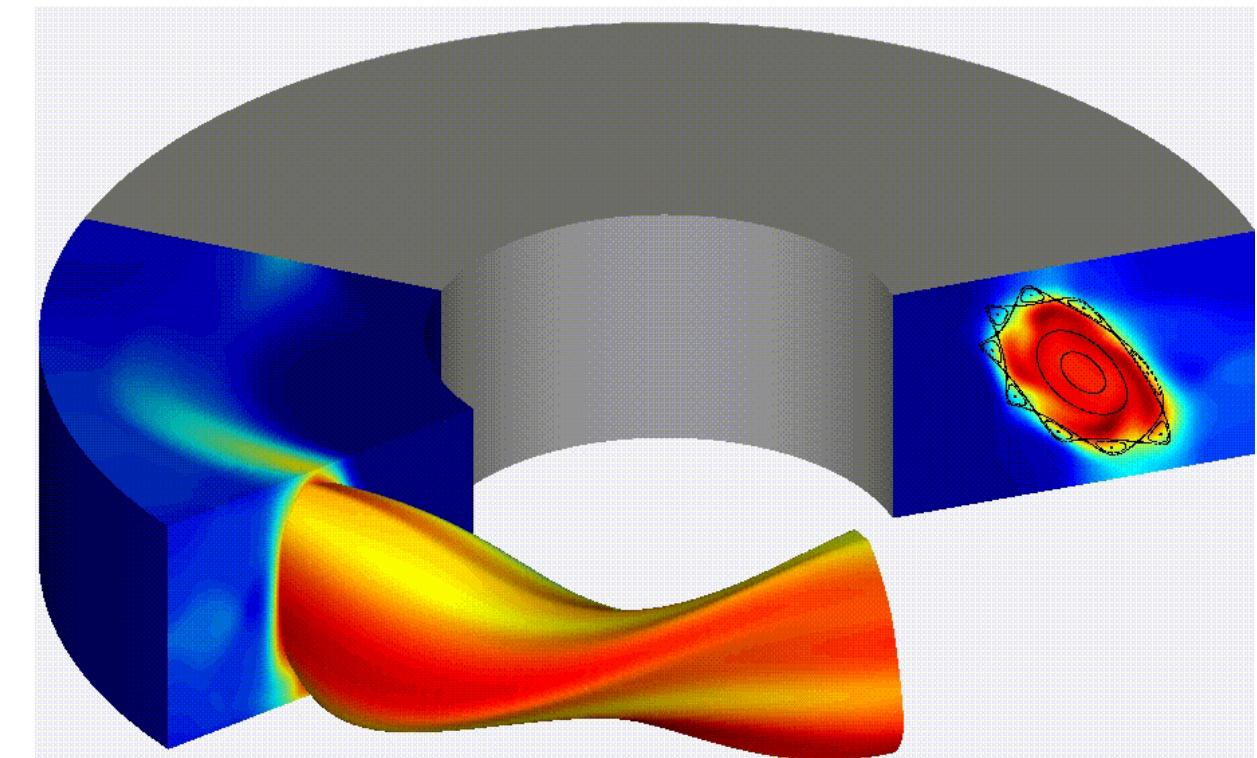
- Electrostatic perpendicular electric field:**  $\mathbf{E}_{\perp} = -\nabla_{\perp} \Phi$  solved with **Poisson**
- Similar equations for  $V_{\parallel,e}, V_{\parallel,i}, T_e, T_i$  given by **momentum and energy conservation**
- Charge conservation**,  $\nabla \cdot (\mathbf{J}_{\parallel} + \mathbf{J}_{di} + \mathbf{J}_{pol}) = 0$  + **Neutral dynamics** (not considered in the present work) [D. Mancini et al *Nucl. Fusion* 2023]

- GBS is a **two fluid, global, flux-driven code to simulate turbulence** evolving such equations in time in a **non-flux-aligned grid**
- **Coupled with a kinetic model for neutral dynamics**  
*[D. Mancini et al Nucl. Fusion 2023]*

TCV-like tokamak

*[B. De Lucca et al EFTC 2025]*

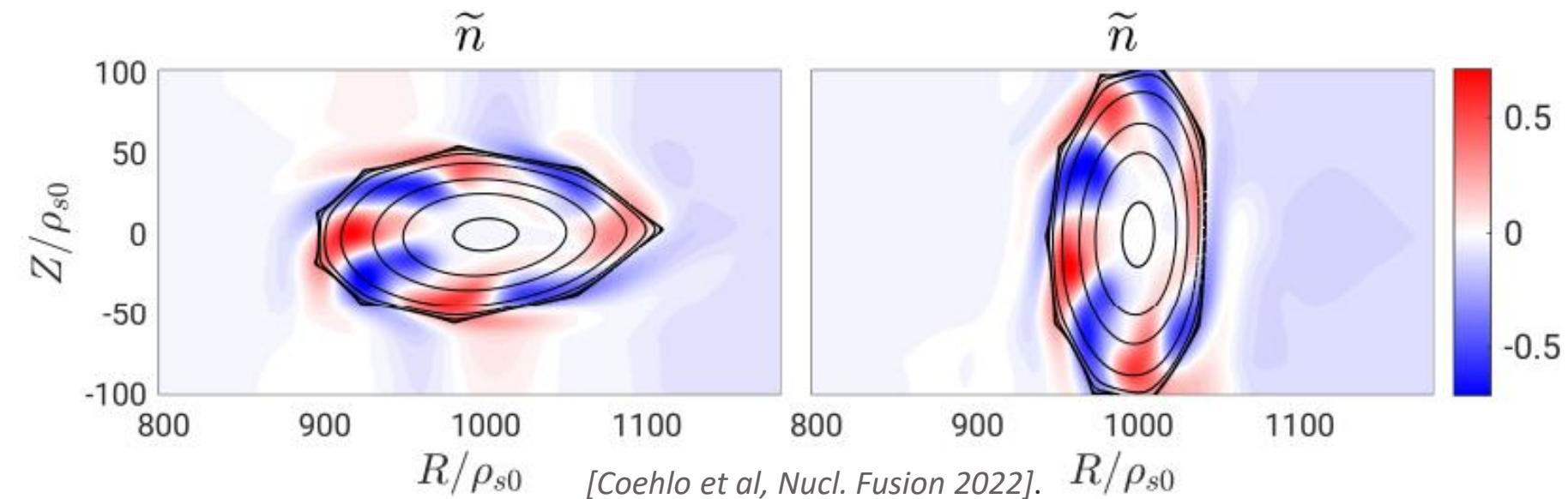
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*[D. Mancini et al *Nucl. Fusion* 2023]*
- GBS used in the past decade to simulate tokamaks *[Ricci et al. PPCF 2012, Giacomin et al. JCP 2022]*; **now it can simulate turbulence in 3D magnetic fields** *[Coehlo et al, *Nucl. Fusion* 2024]*.



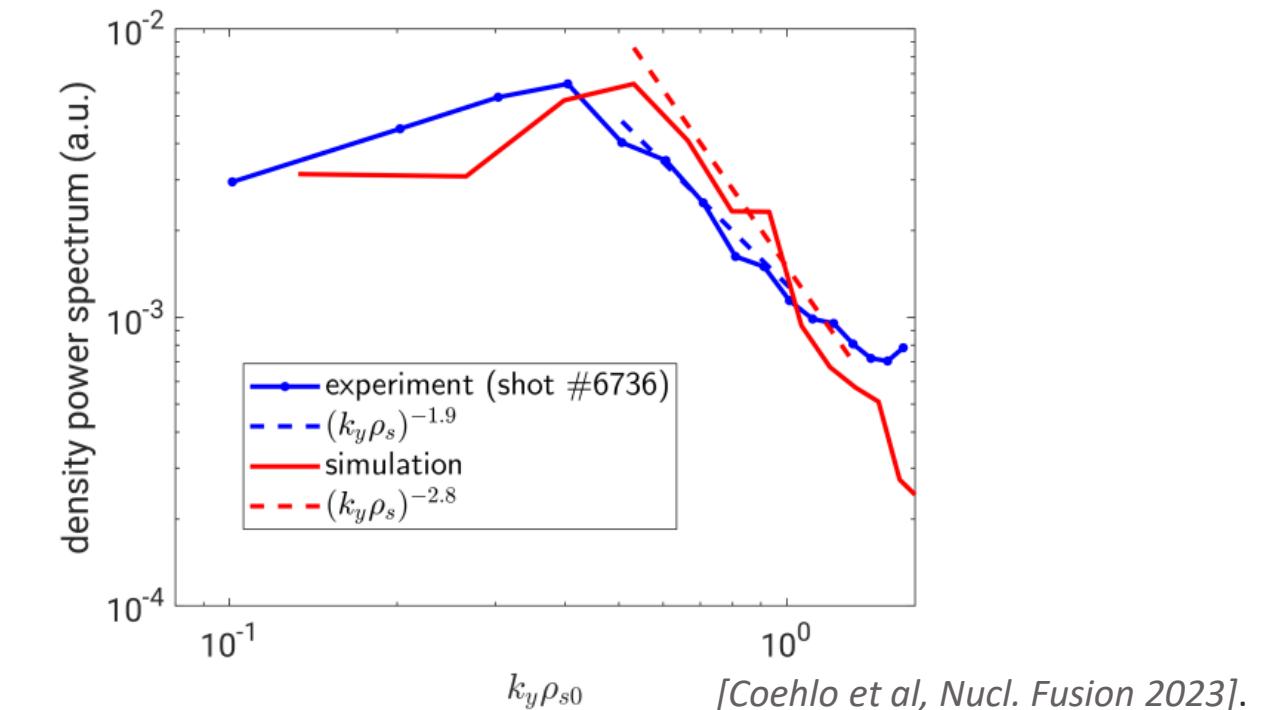
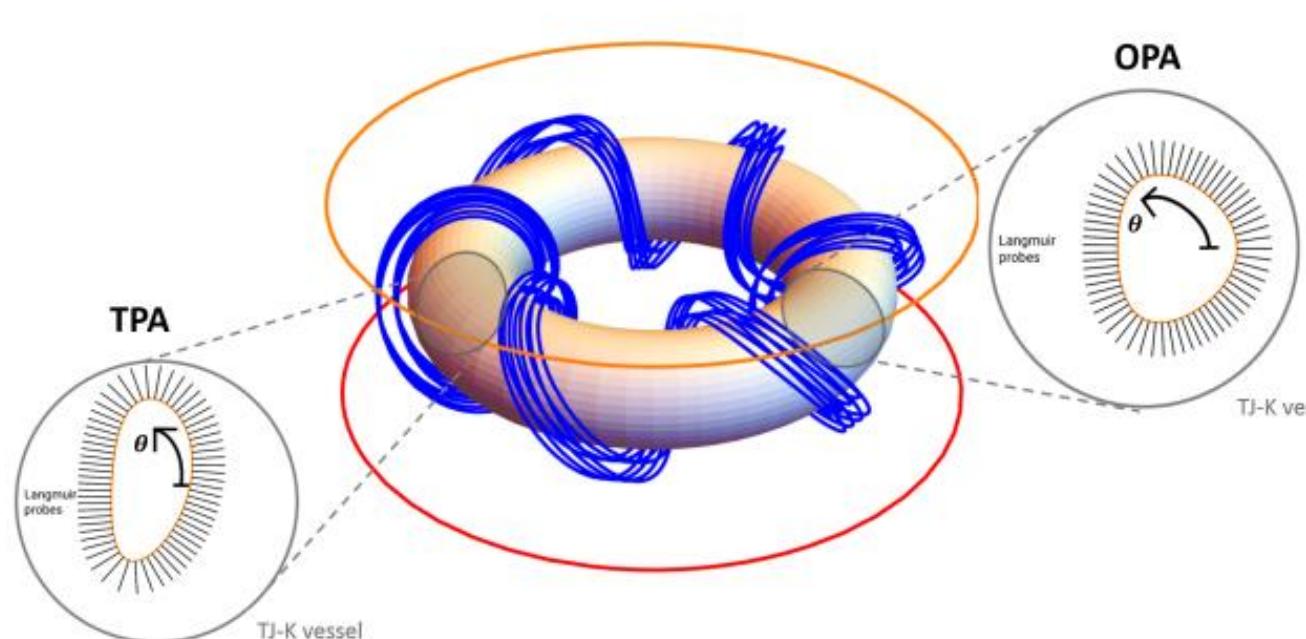
*[Coehlo et al, *Nucl. Fusion* 2024]*

# First-principles edge transport pioneered by the GBS group

- First global flux driven simulation of a toy model stellarator

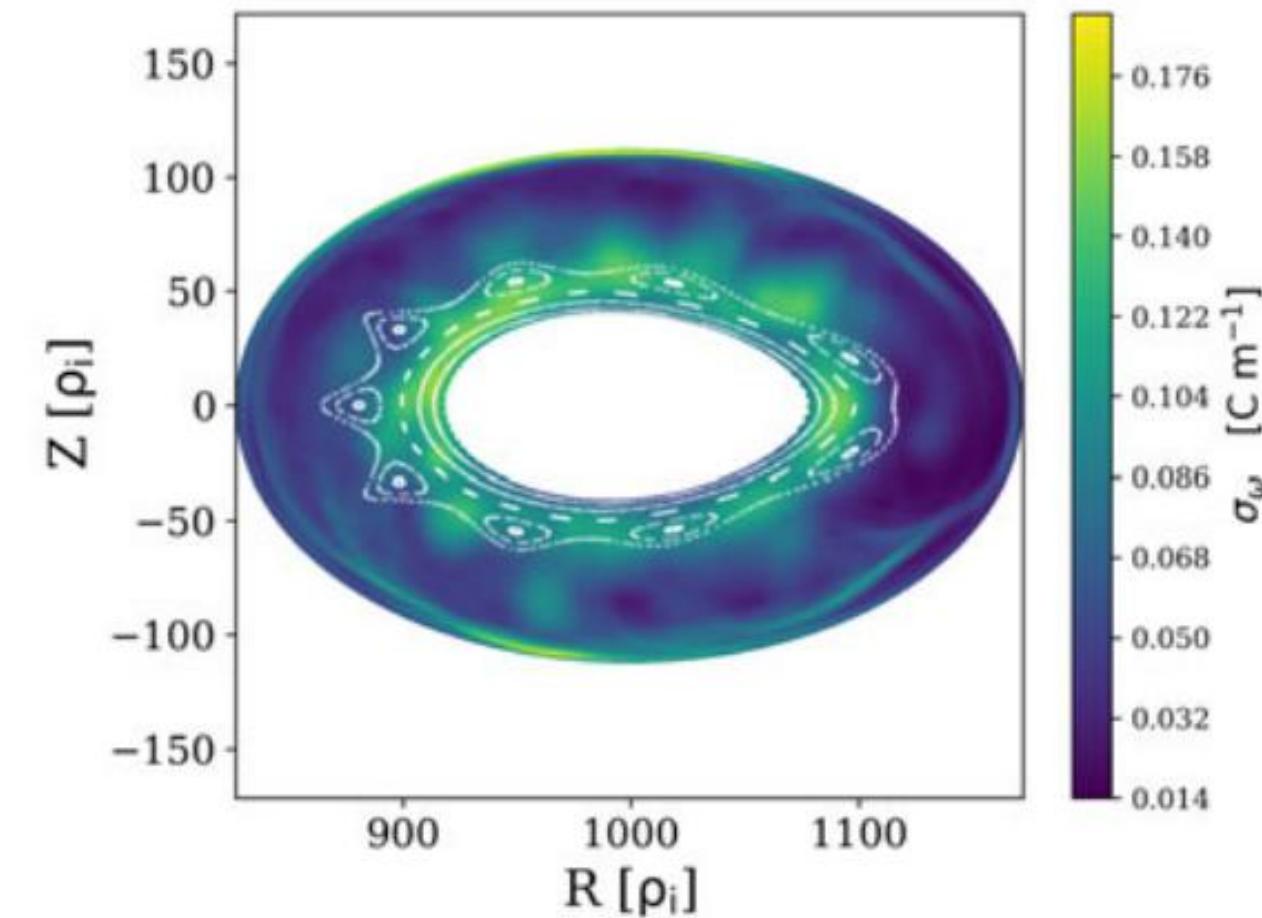


- Successful global quantitative validation in TJK



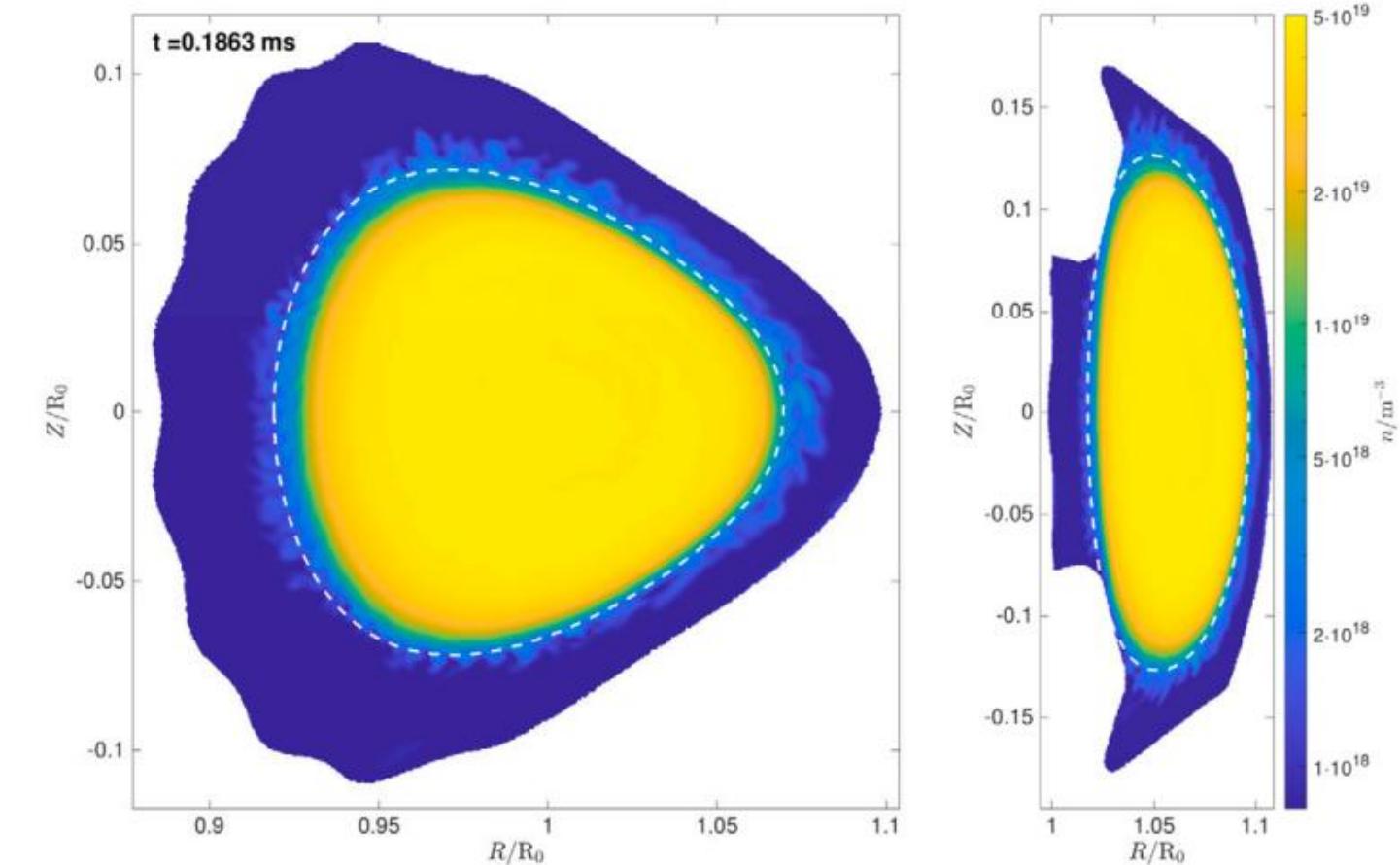
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  - Isothermal fluid model in BOUT++



[B Shanahan et. al., JPP 2024]

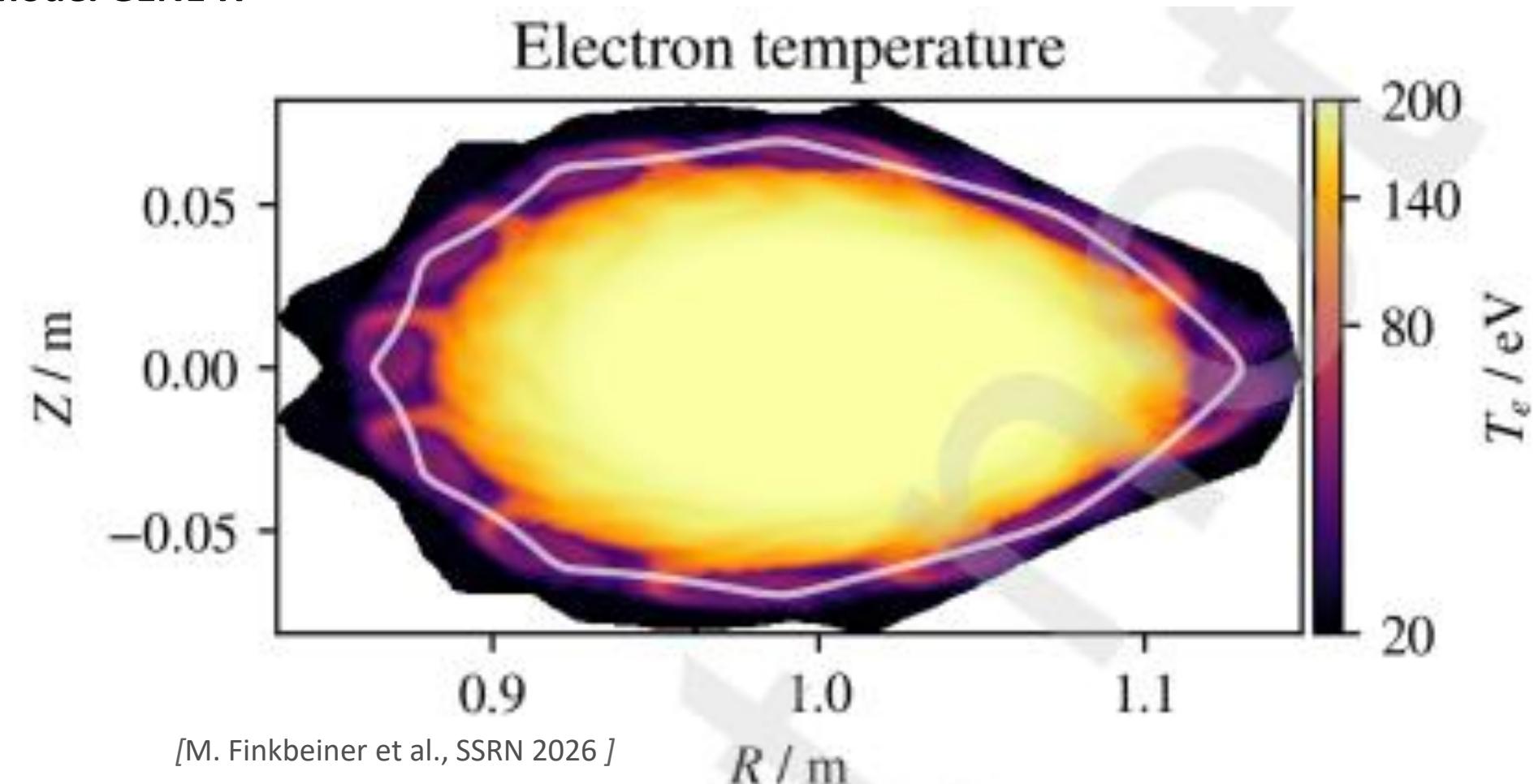
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  - Plasma relaxation fluid model GRILLIX



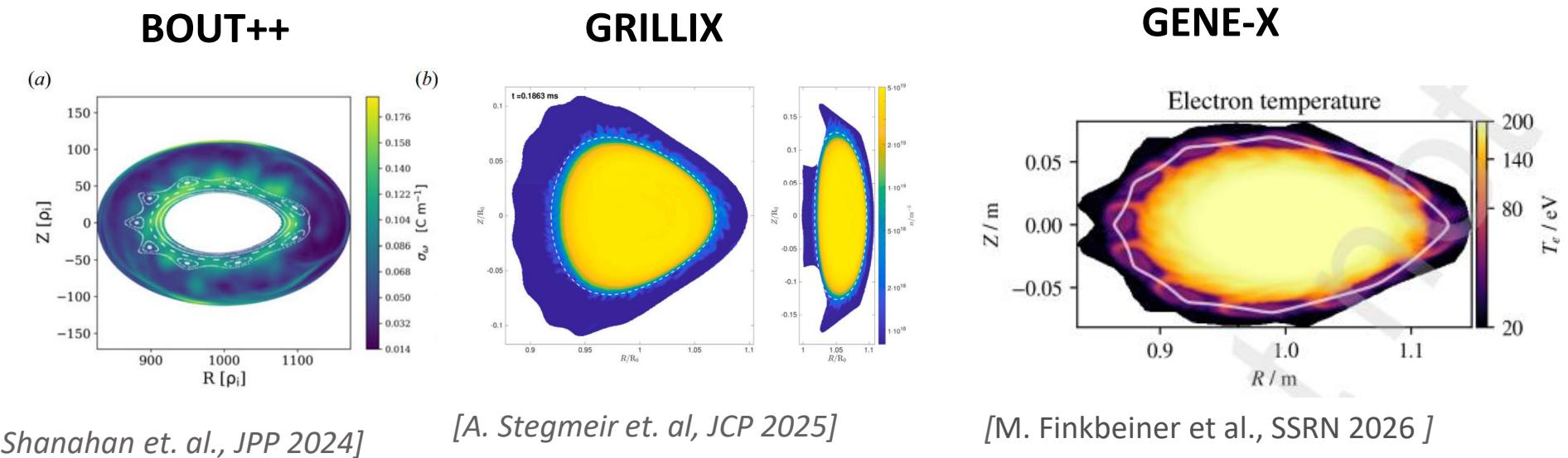
[A. Stegmeir et. al, JCP 2025]

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- Isothermal fluid model in **BOUT++**
  - Plasma relaxation fluid model **GRILLIX**
  - Plasma relaxation kinetic model **GENE-X**

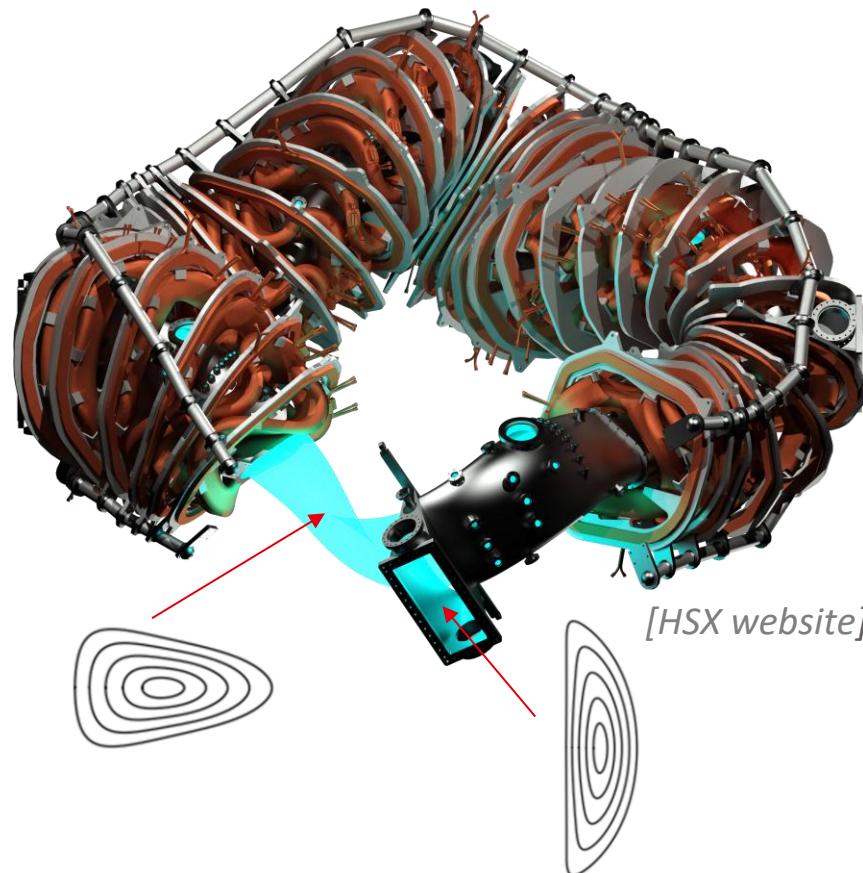


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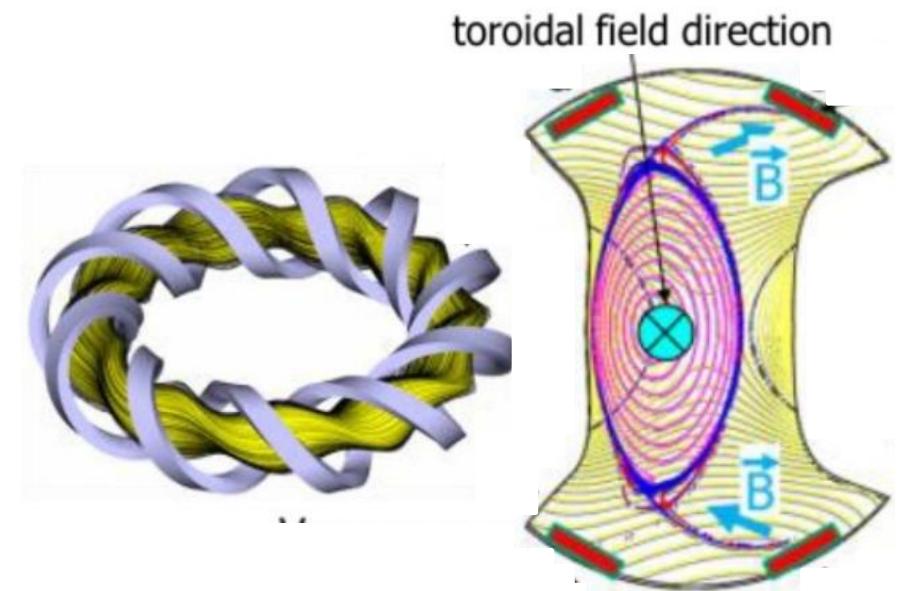
- Here we **examine and validate fusion-relevant machines**, to **understand the impact of three-dimensional geometries on edge turbulence**

HSX



[HSX website]

LHD

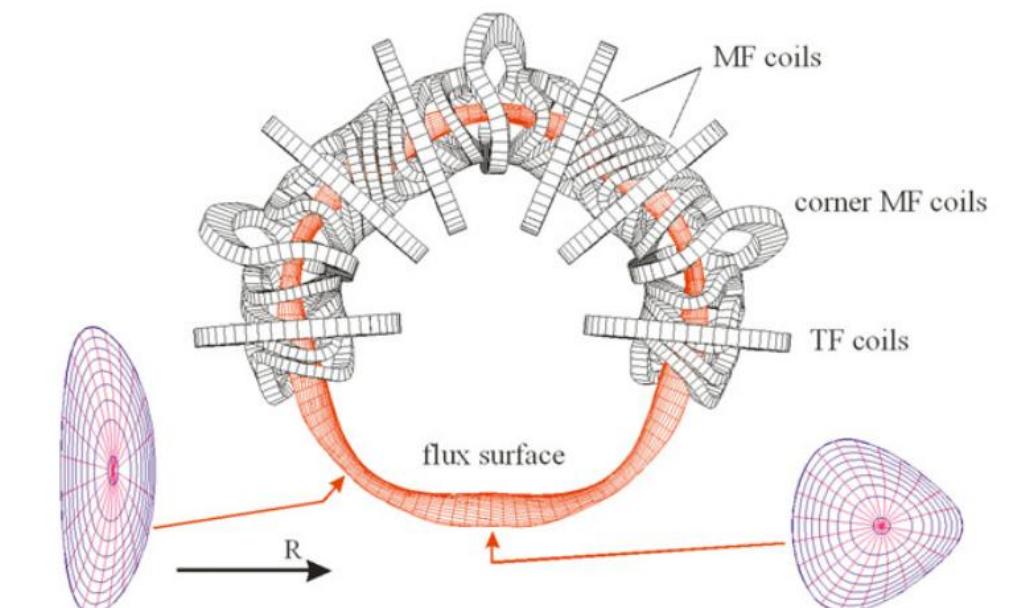


[S Masuzaki et al. Nuclear Materials and Energy 2019]

Largest **heliotron** stellarator in the world  
Stochastic layer, X-point divertors

Only **quasi-helically stellarator** in the world

W7-AS

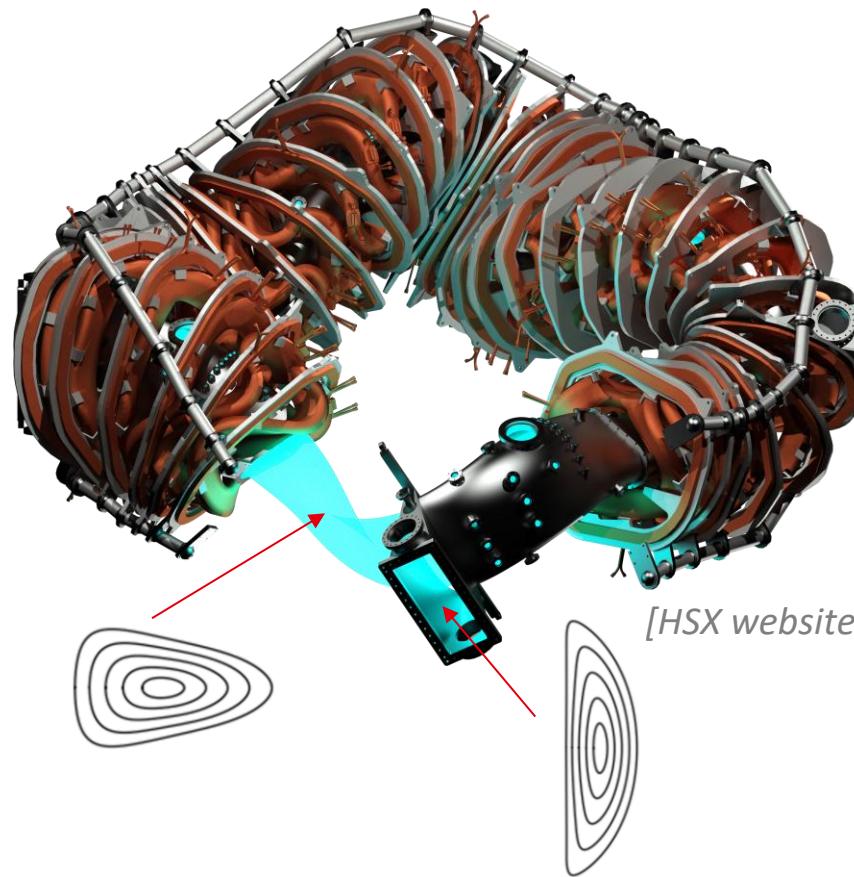


[Hirsch et al, PPCF 2008]

Predecessor of W7-X sharing many features  
but smaller in size

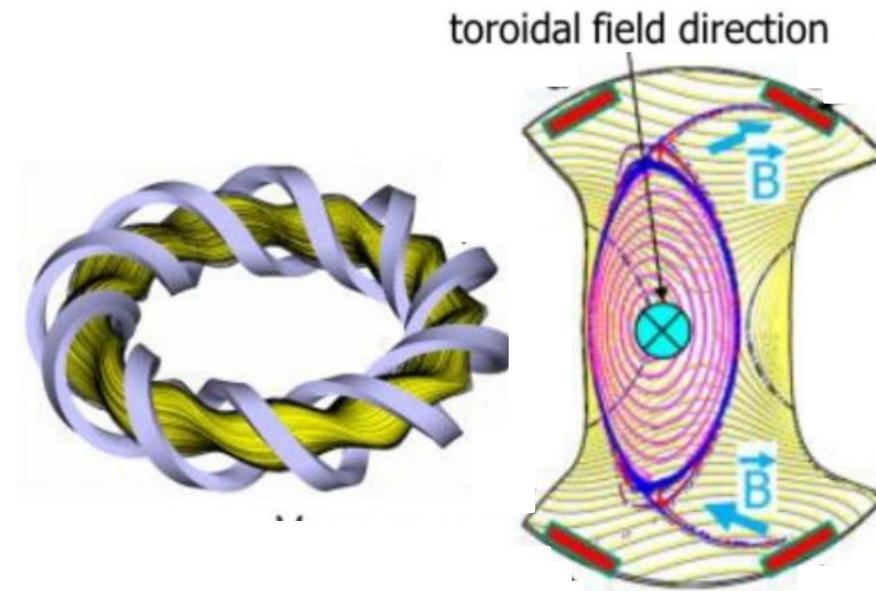
Auxiliary coil currents allowed **great flexibility**  
in the edge topology

HSX



[HSX website]

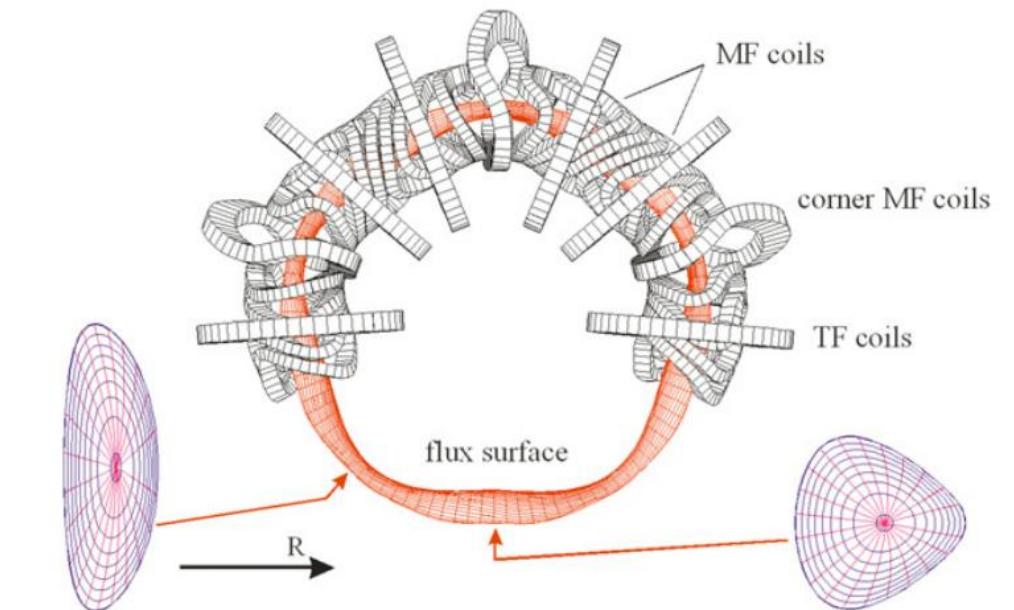
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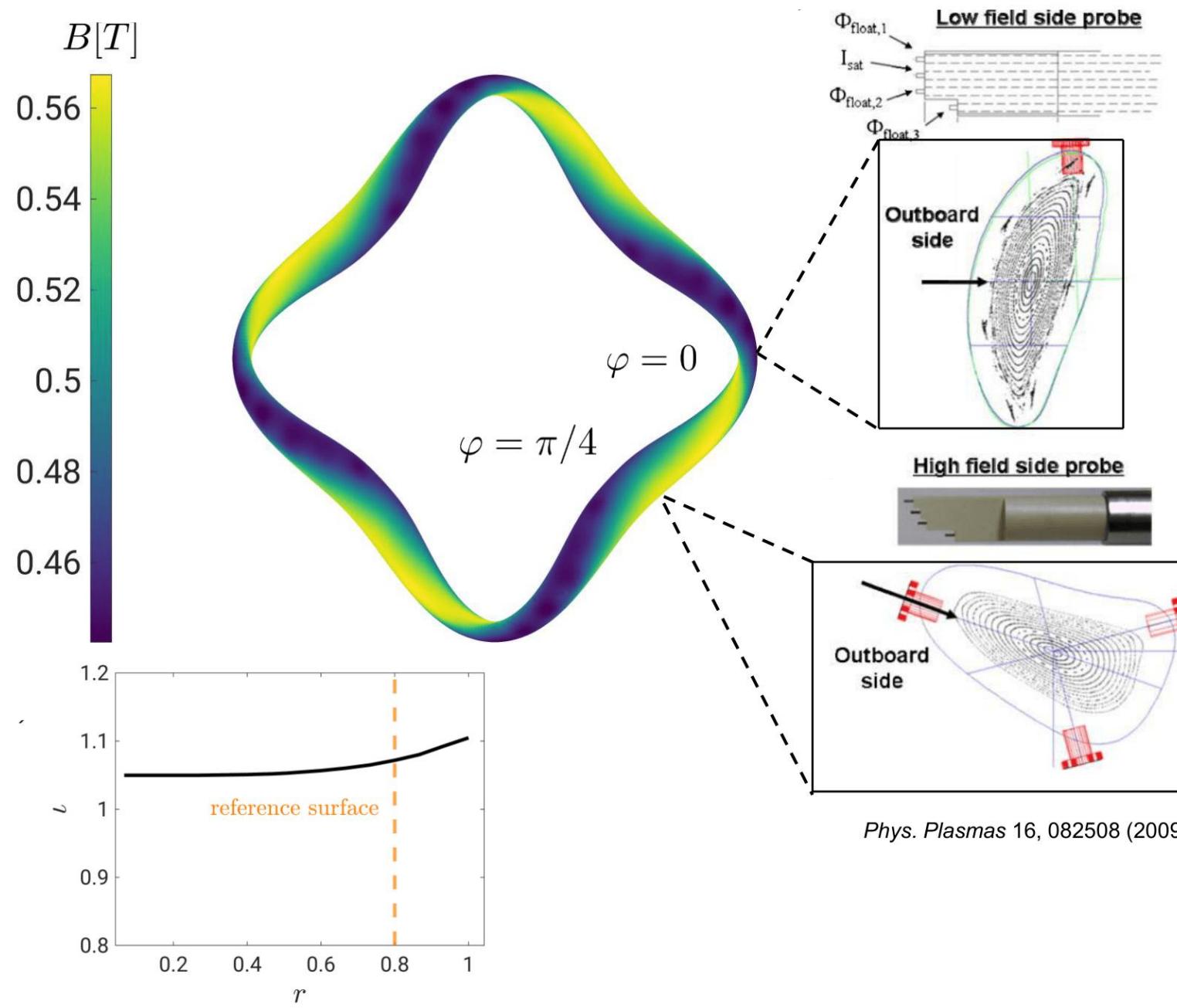


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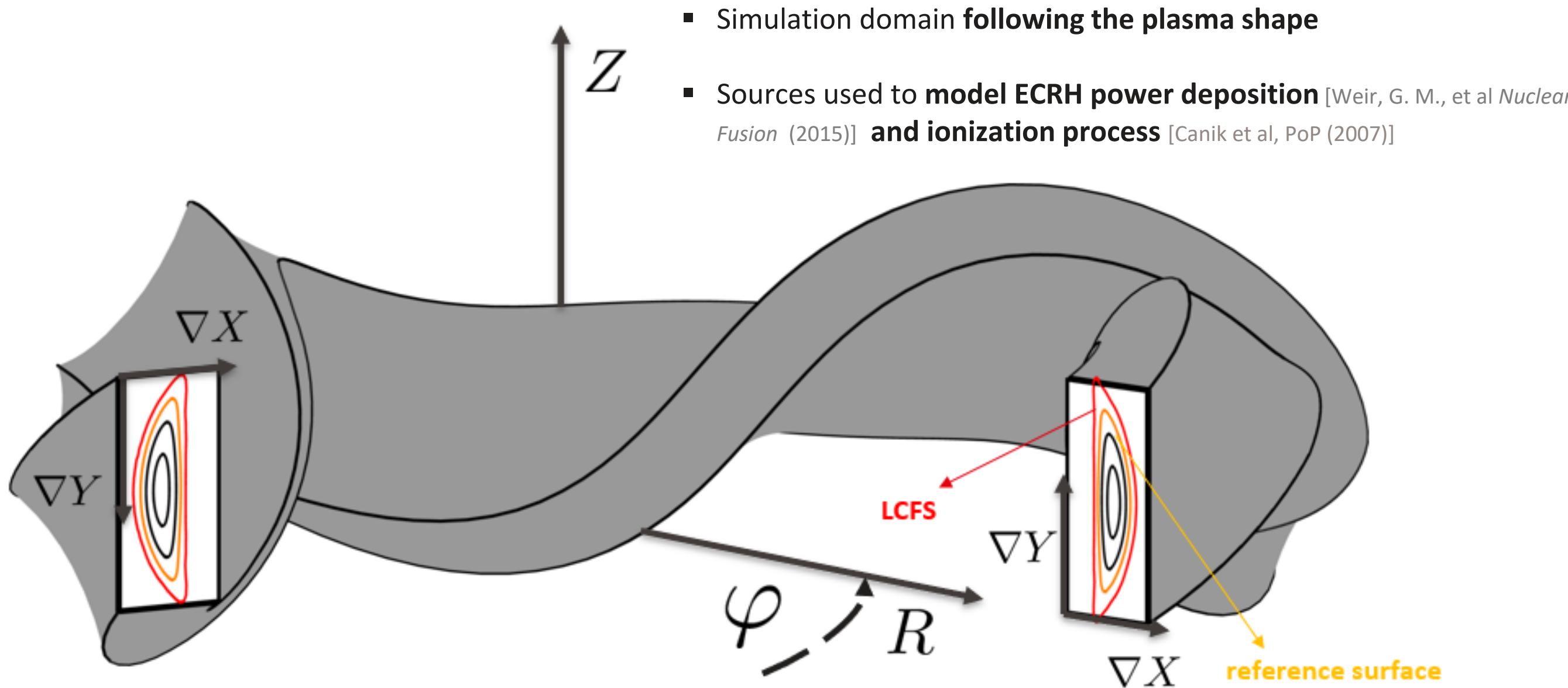
- Simulate full-size machine with comparison to experiments done by **W. Guttenfelder 2009**
- Langmuir probes and Thomson scattering diagnostic
- Turbulent properties compared at the **reference surface**

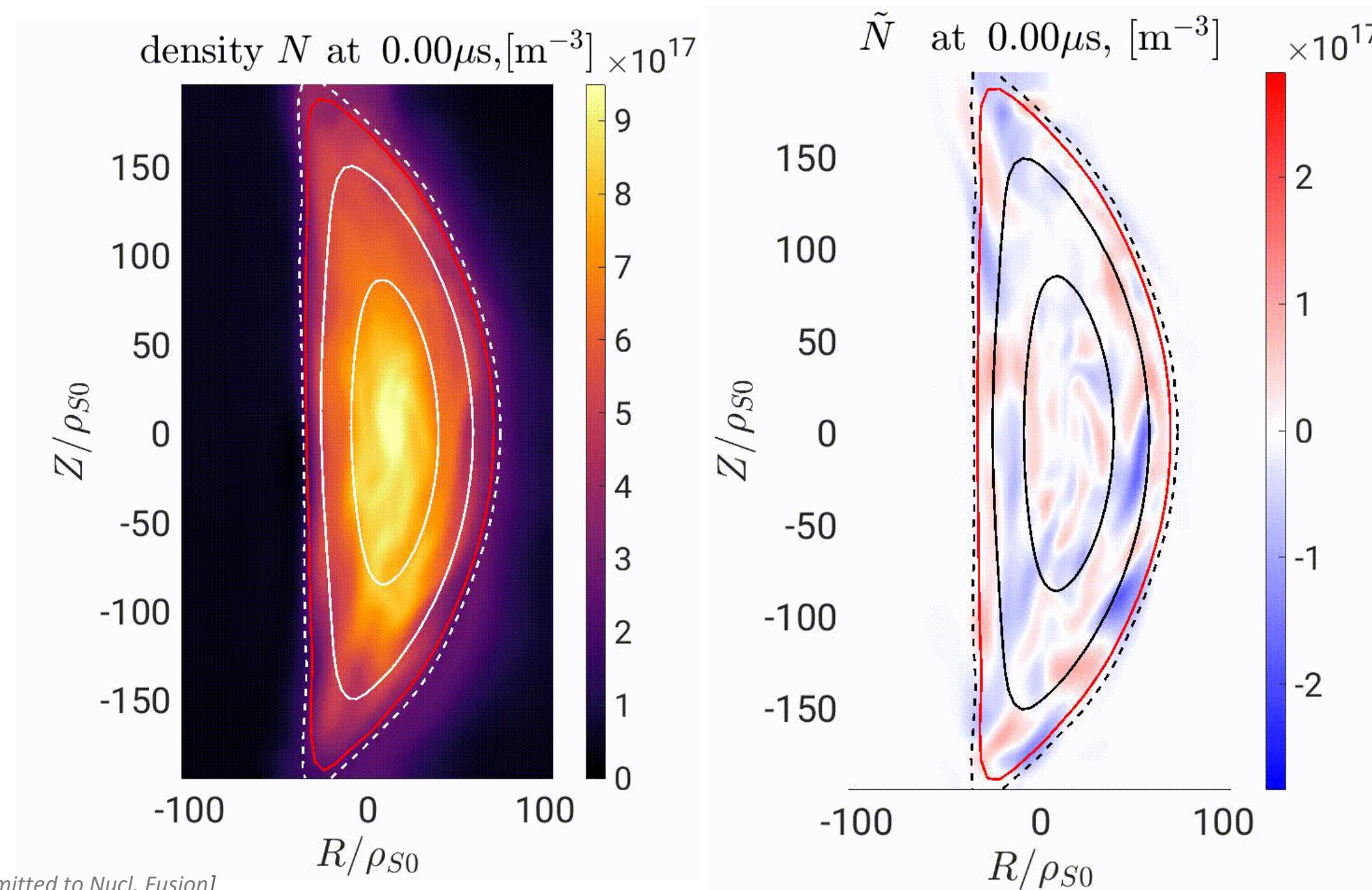
$$B_0 = 0.5 \text{ T}$$

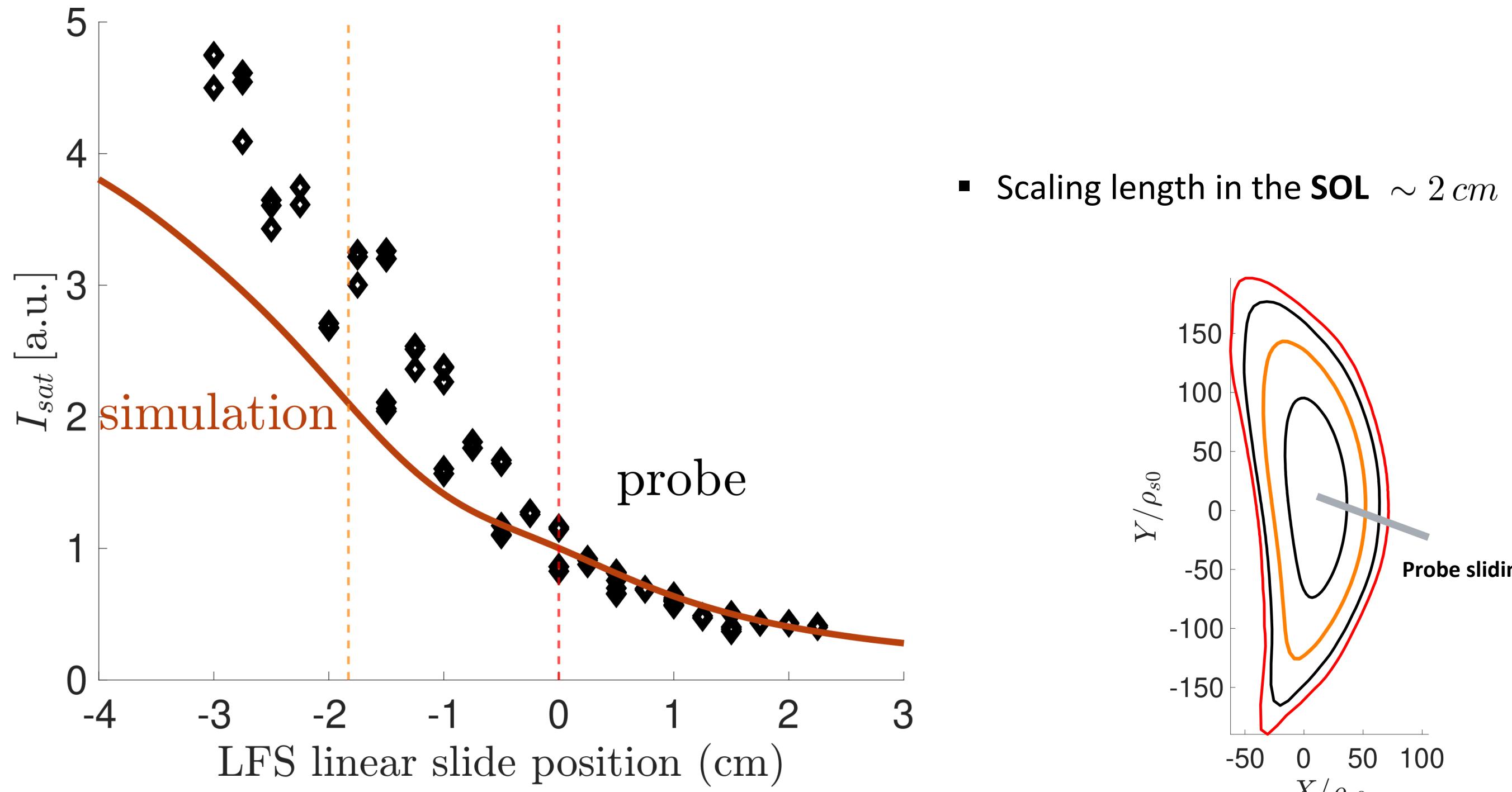
$$n_0 = 10^{18} \text{ m}^{-3}$$

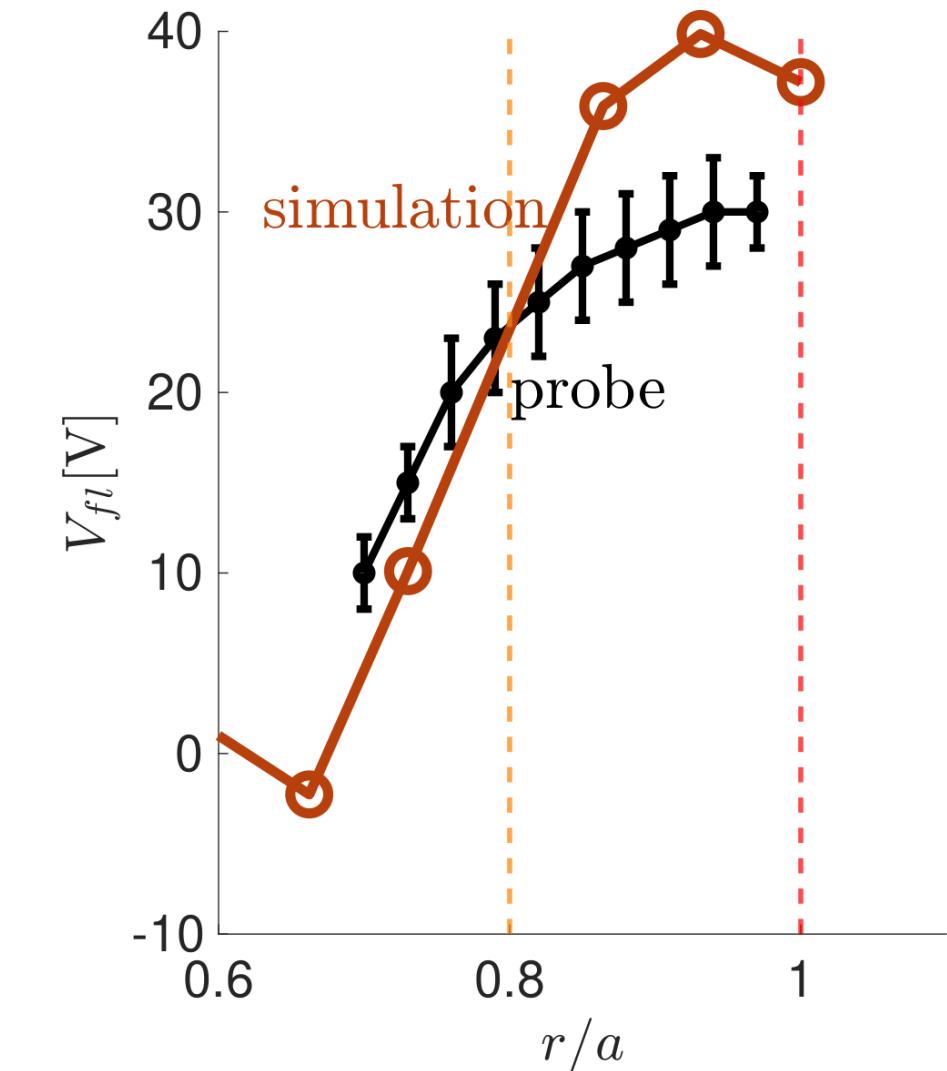
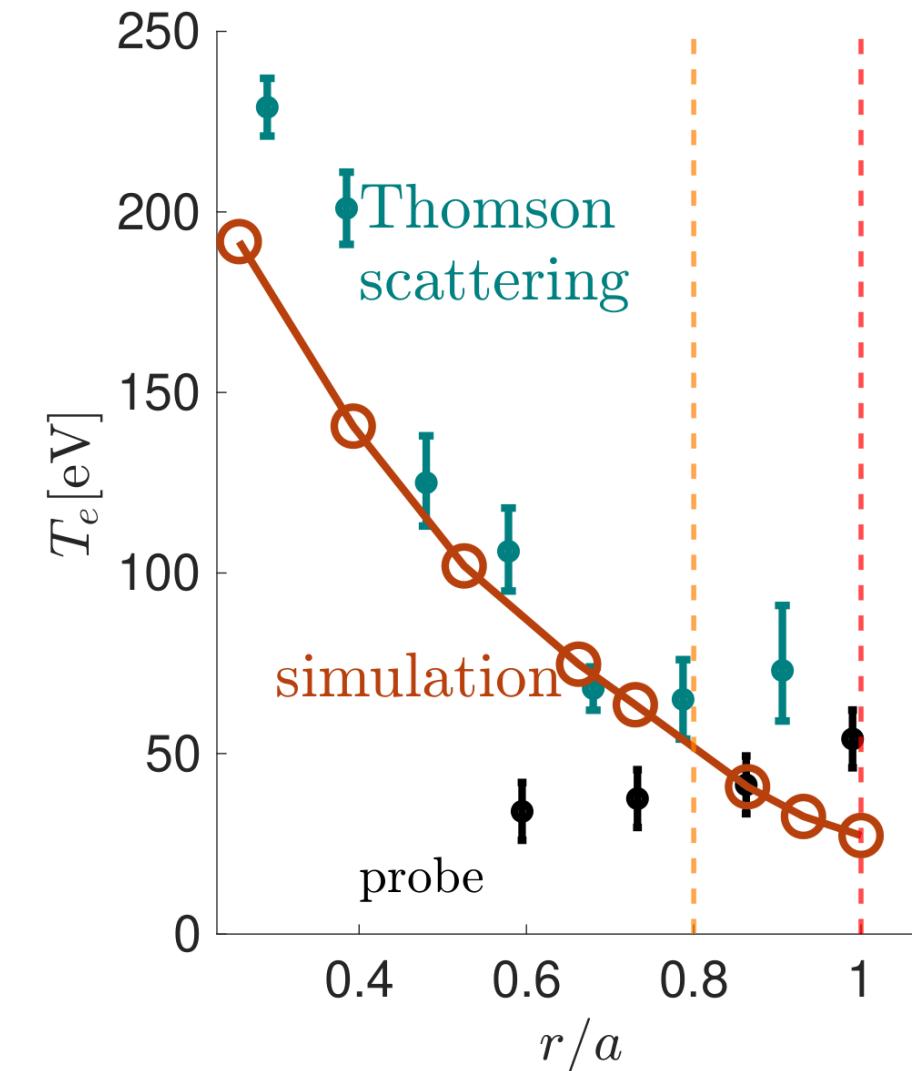
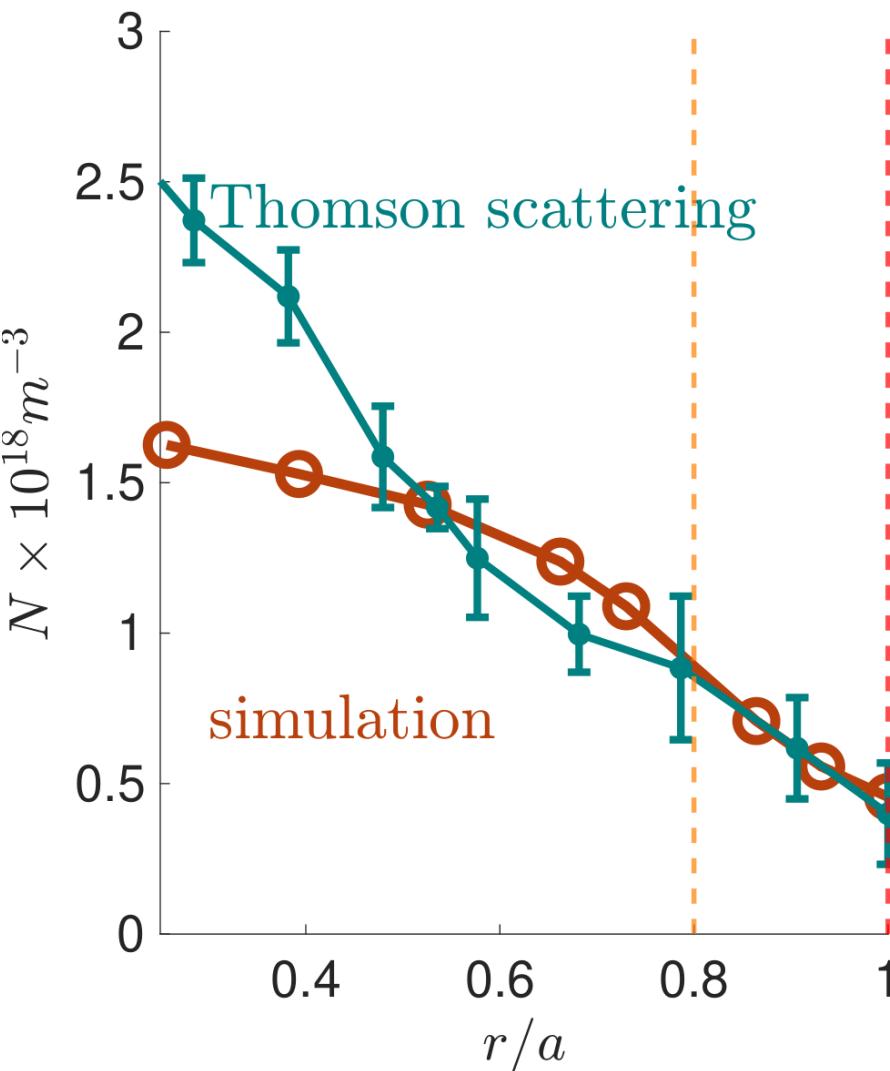
$$T_e = 40 \text{ eV}$$

$$T_i = 20 \text{ eV}$$

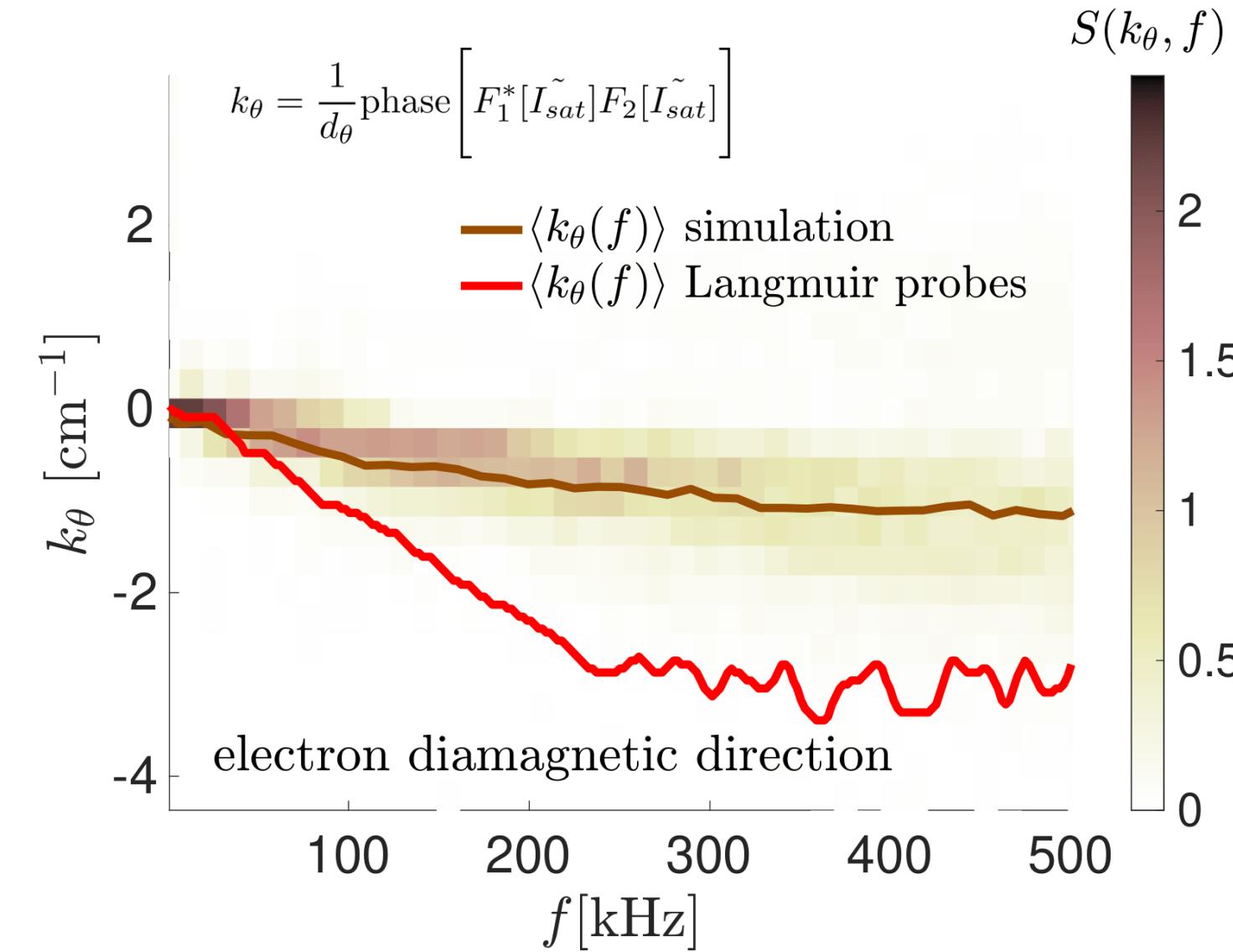
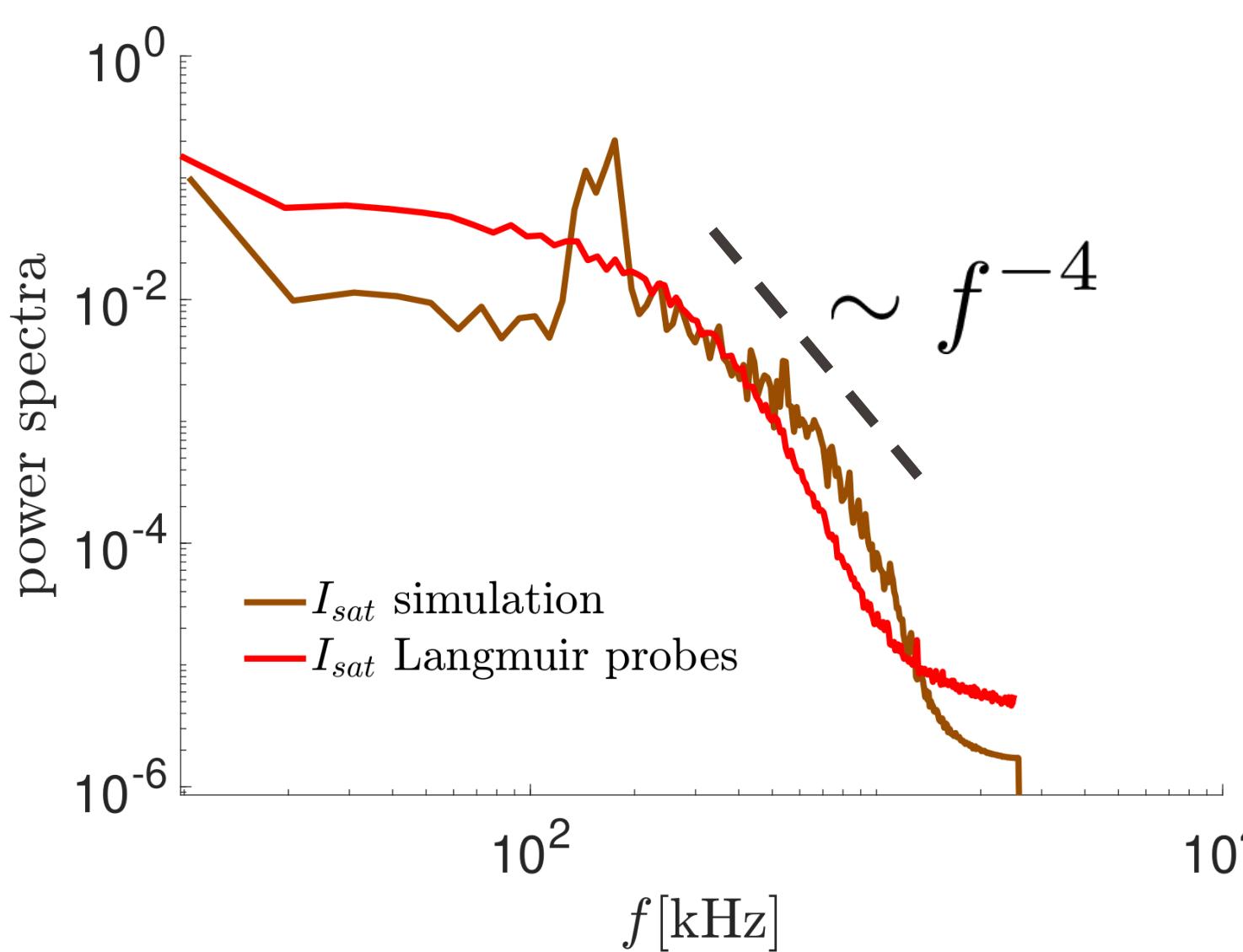




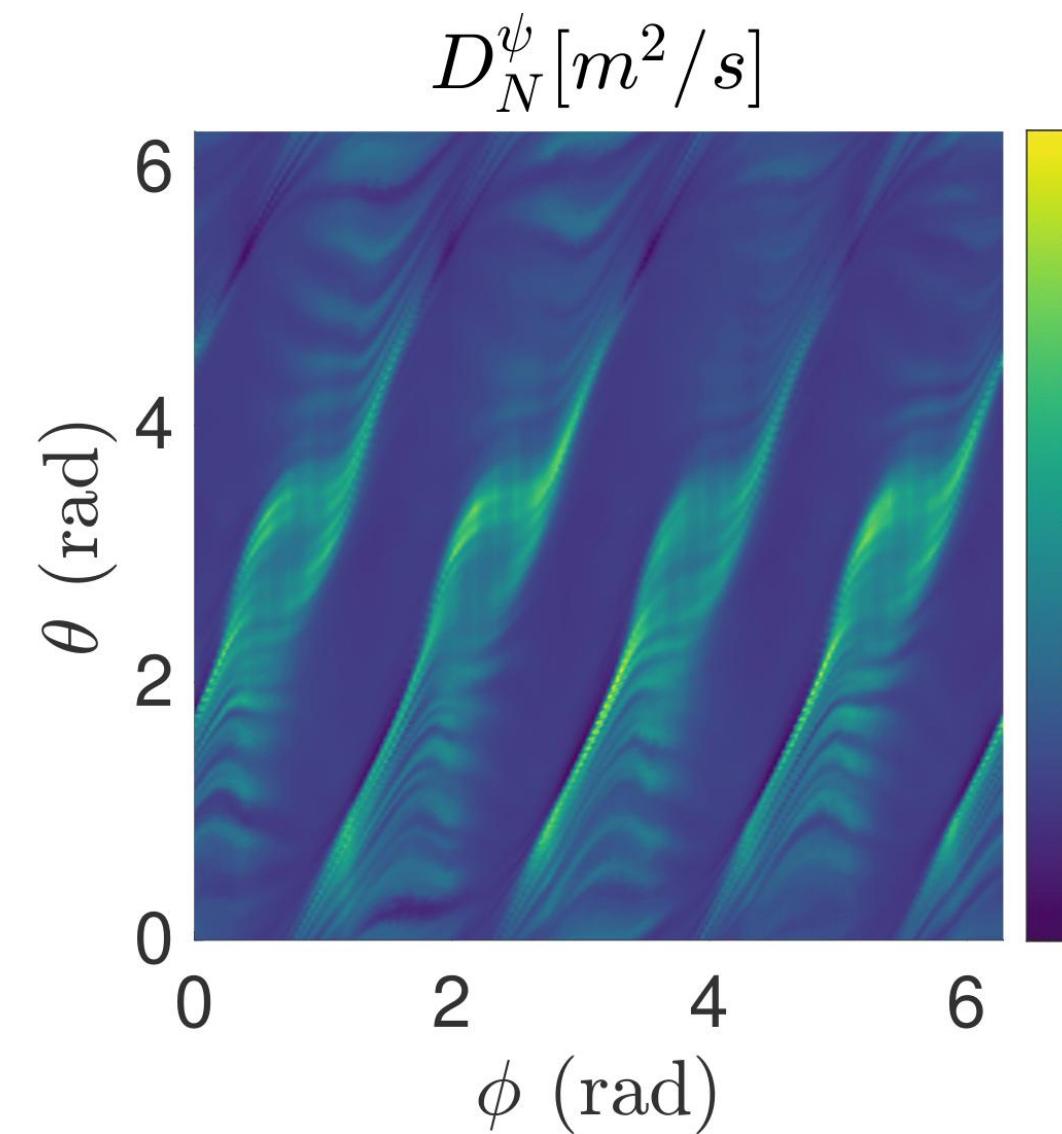




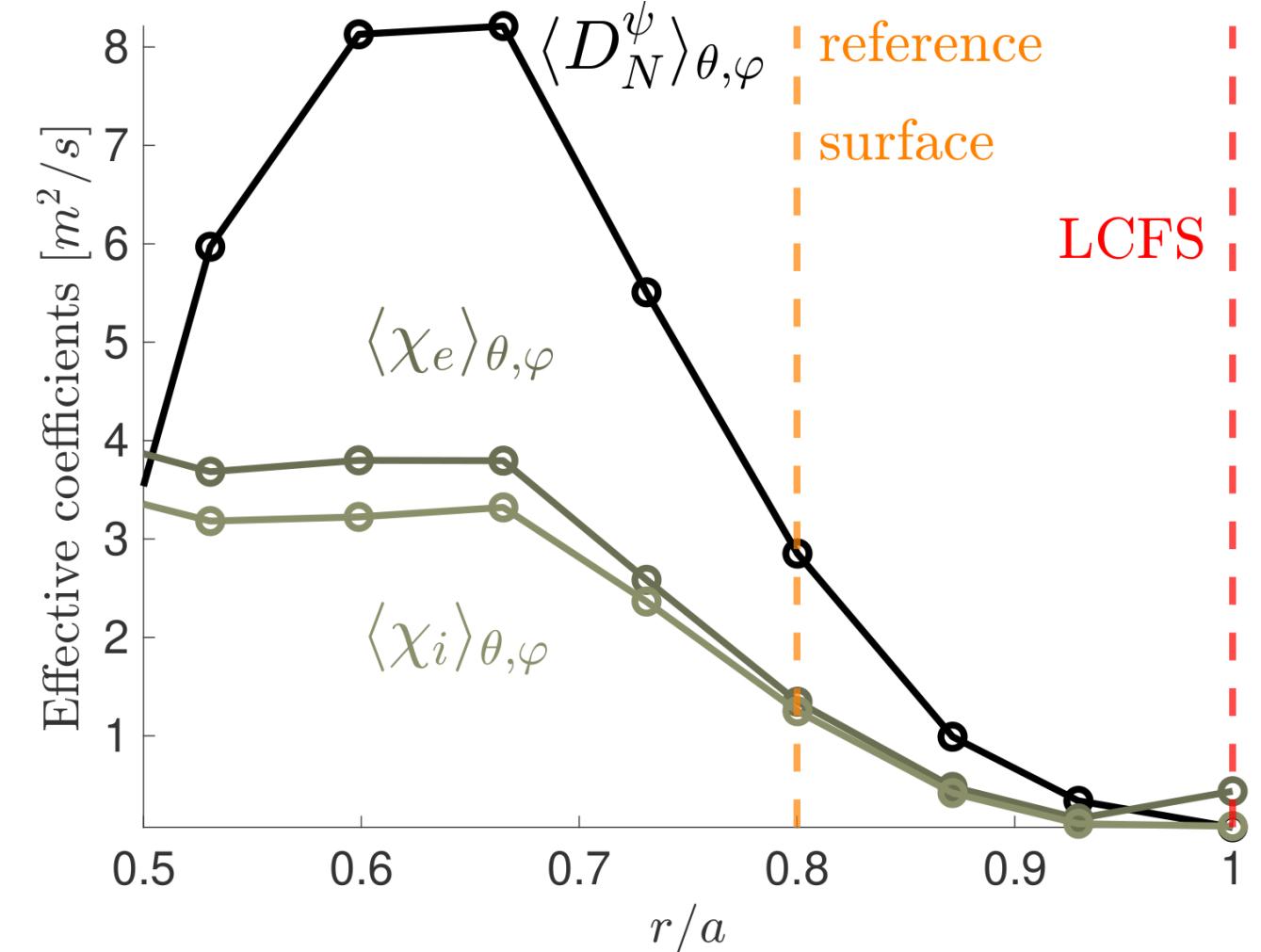
- Negative edge radial electric field



- **More energy** around  $f \approx 200 \text{ kHz}$  **with cascade of**  $\sim f^{-4}$
- **Poloidal correlation**  $\rho_s \langle k_\theta \rangle \approx 0.12$  and  $L_\theta \approx 6 \rho_s$  **consistent with measurements of**  $\rho_s \langle k_\theta \rangle \approx 0.14$  and  $L_\theta \approx 7 - 8$



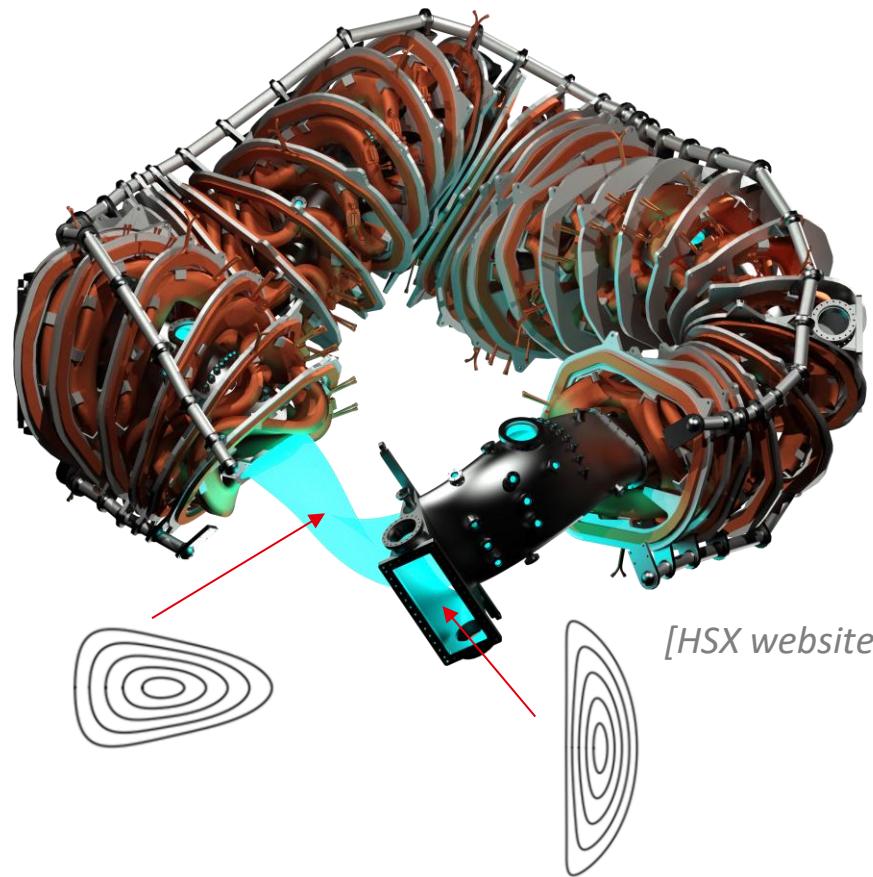
- Regions of negative particle transport



$$D_N^\psi = -\langle \tilde{\Gamma}_N^\psi \rangle_t / \langle \nabla N \cdot \mathbf{n}^\psi \rangle_t$$

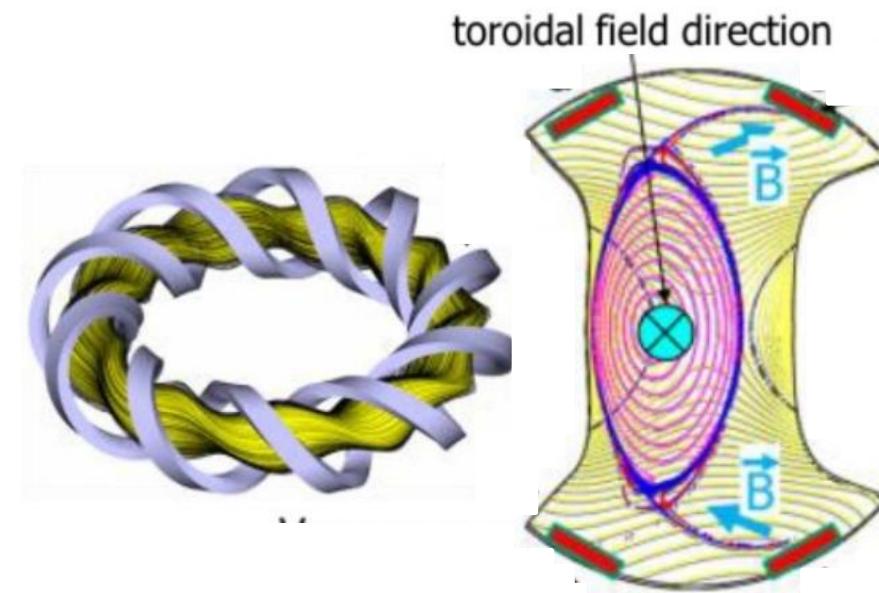
$$\chi_{e,i}^\psi = -\langle \tilde{T}_{i,e} \tilde{V}_{E \times B}^\psi / \nabla T_{i,e} \cdot \mathbf{n}^\psi \rangle_t$$

HSX



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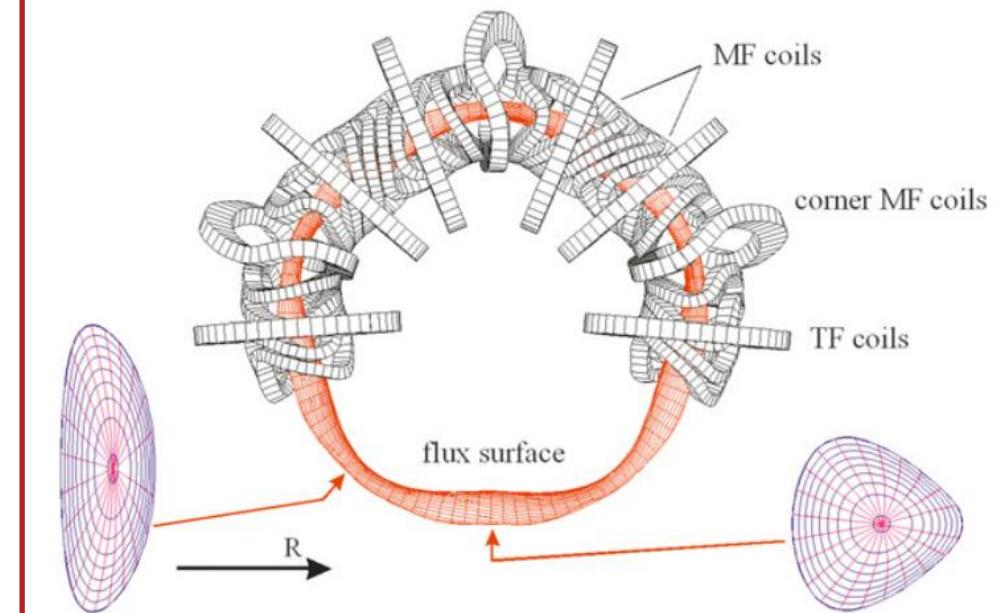
LHD



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W7-AS



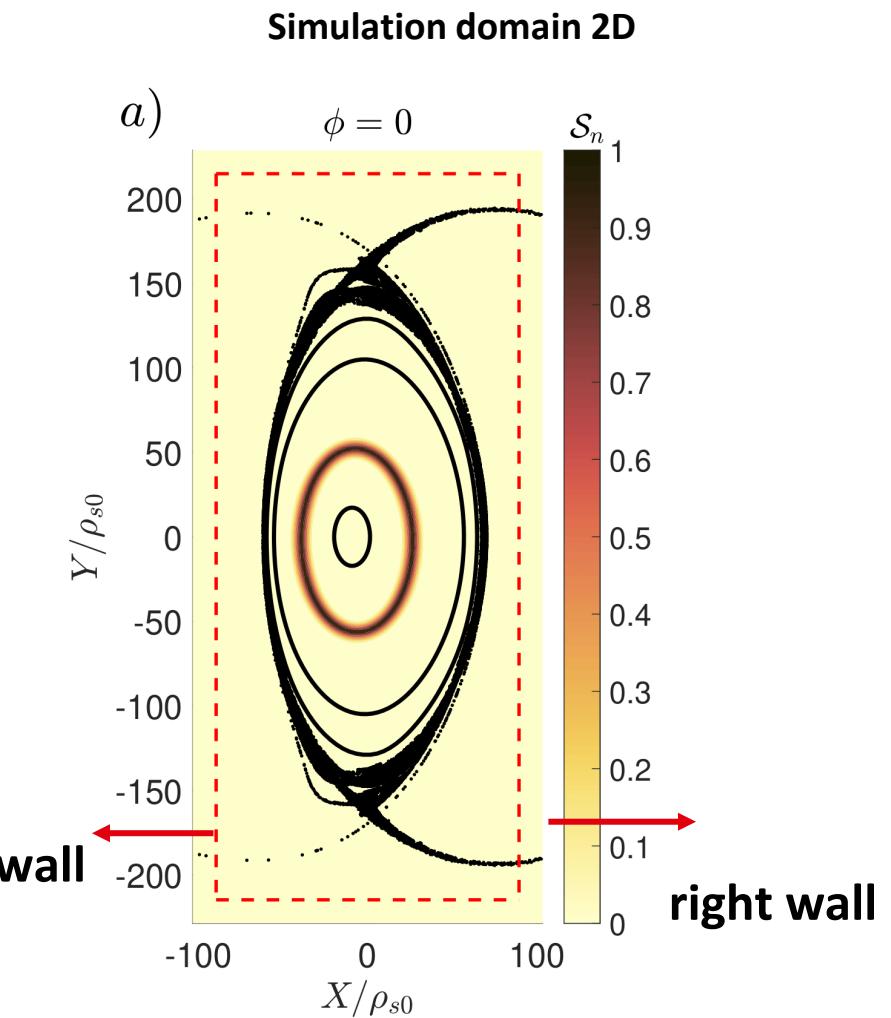
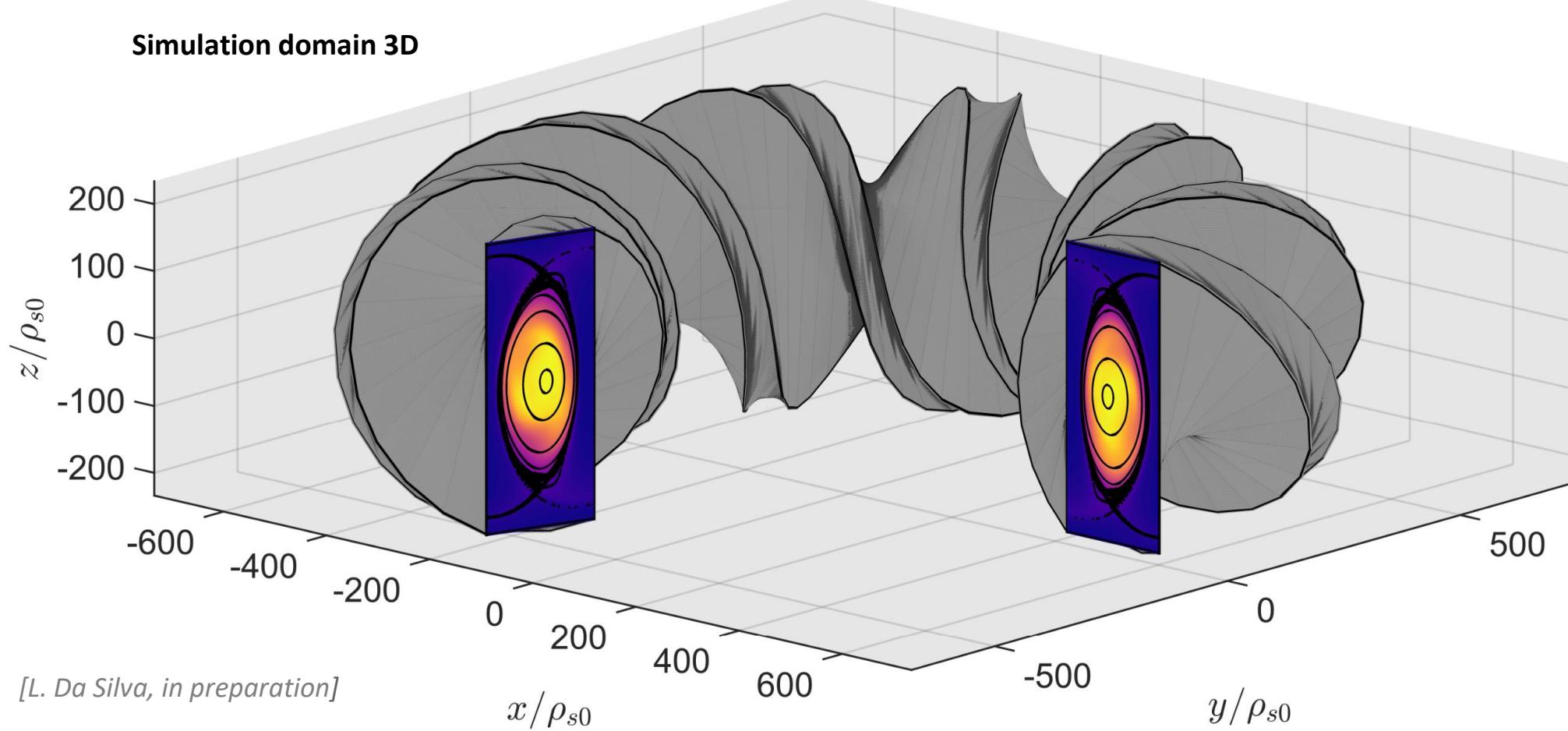
[Hirsch et al, PPCF 2008]

Predecessor of **W7-X** sharing many features  
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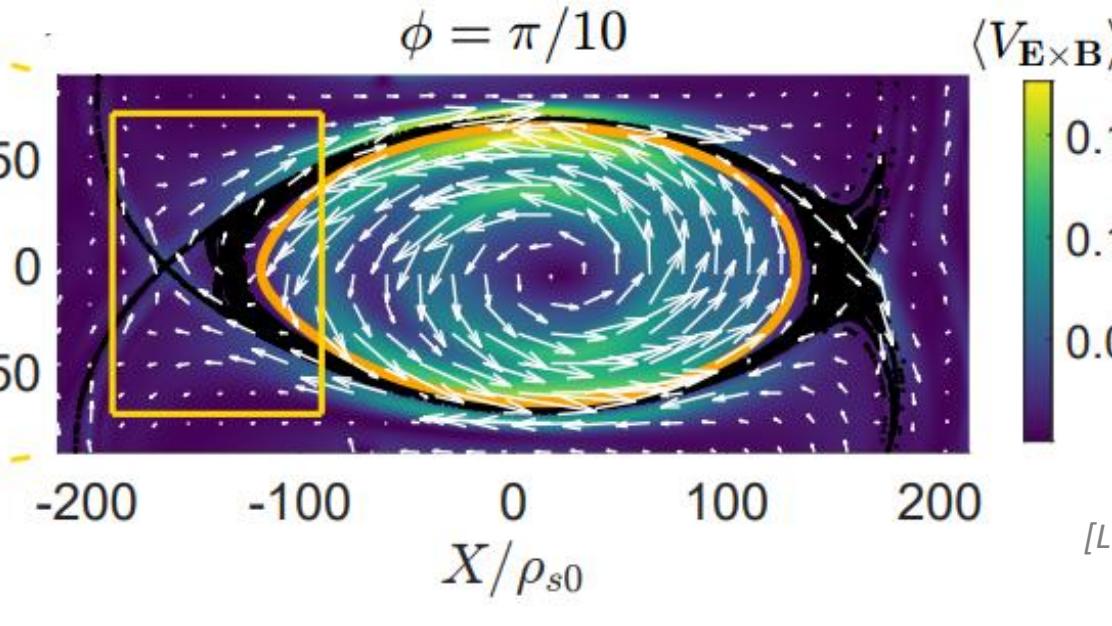
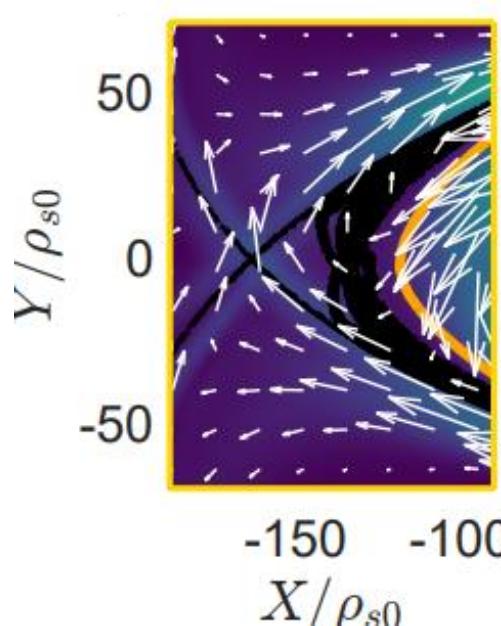
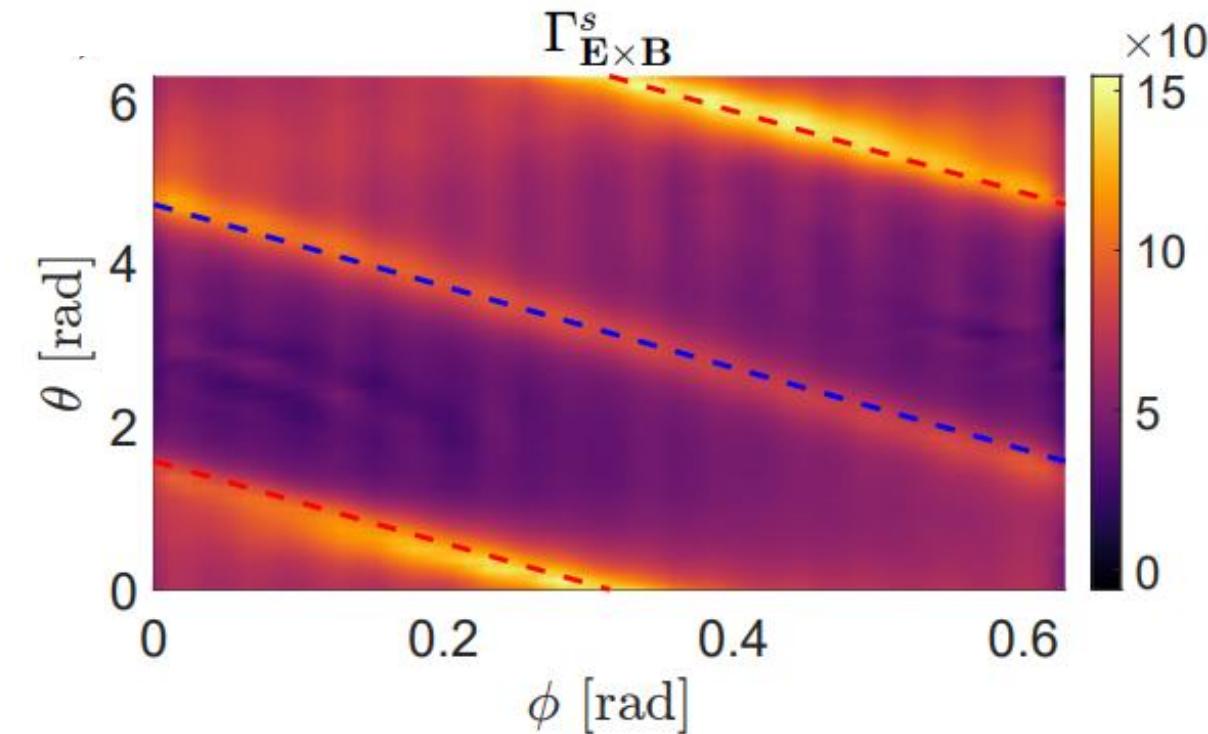
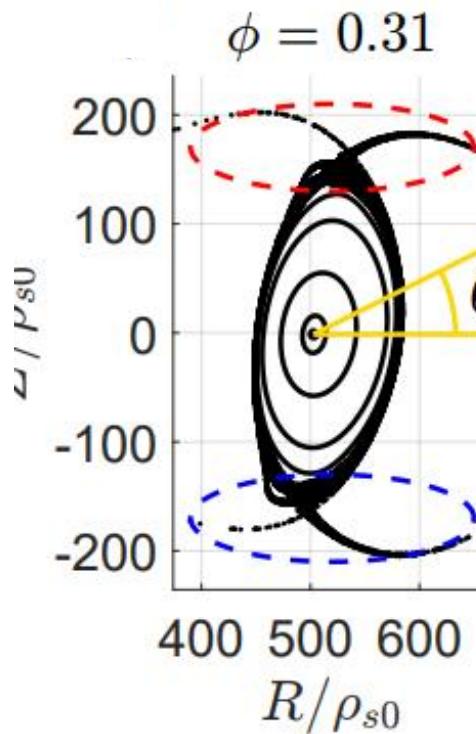
Auxiliary coil currents allowed **great flexibility**  
in the edge topology

**EPFL** Simulating LHD for studying plasma dynamics in the chaotic divertor

- Simulation domain follows **the divertor plates geometry**
- Considering **1/3 of real size** for gaining computational time
- **Qualitative comparison** with experimental results

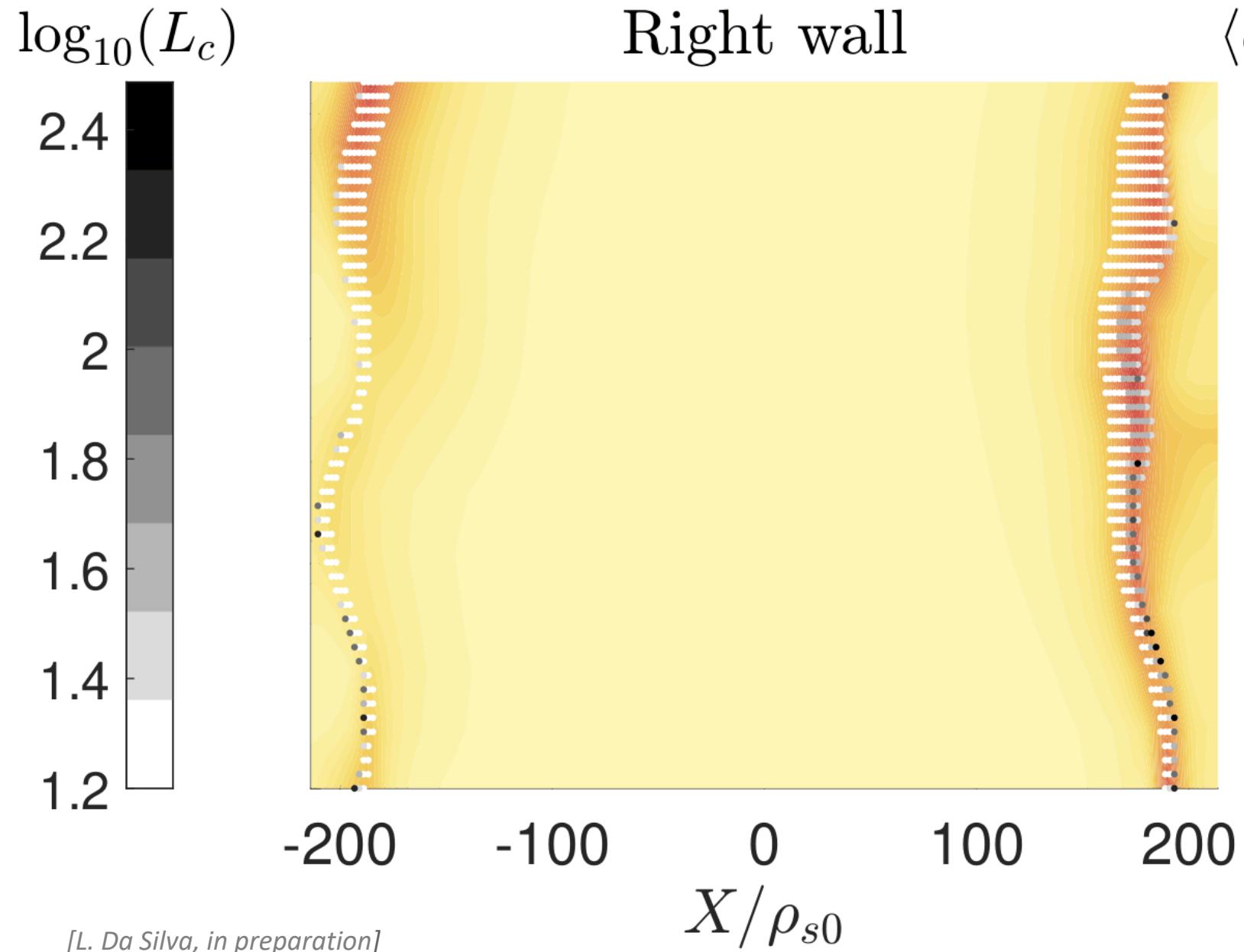


# Self-consistent drifts produce asymmetries in divertors profiles

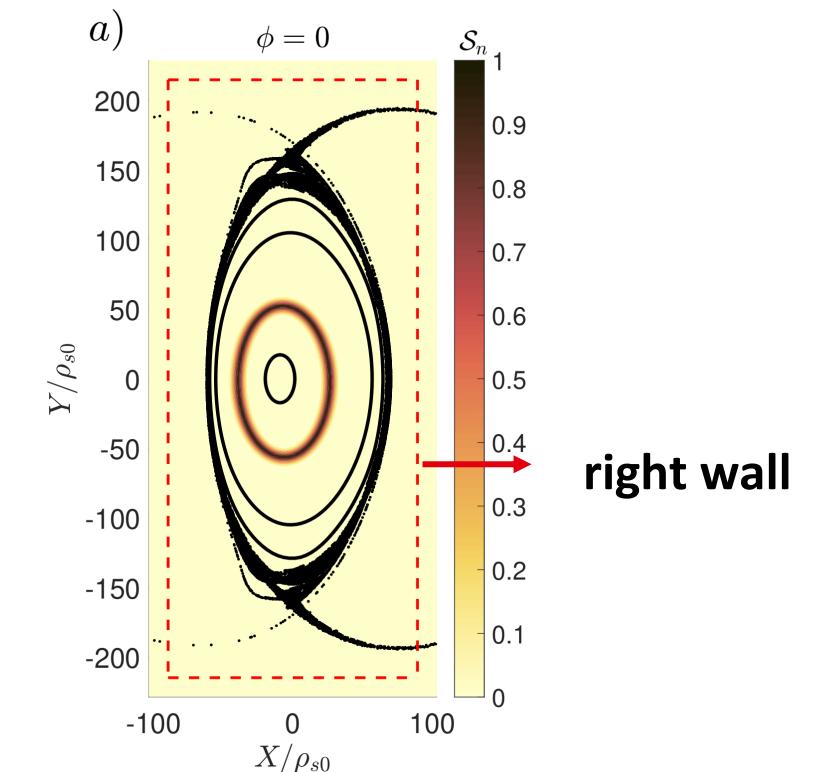


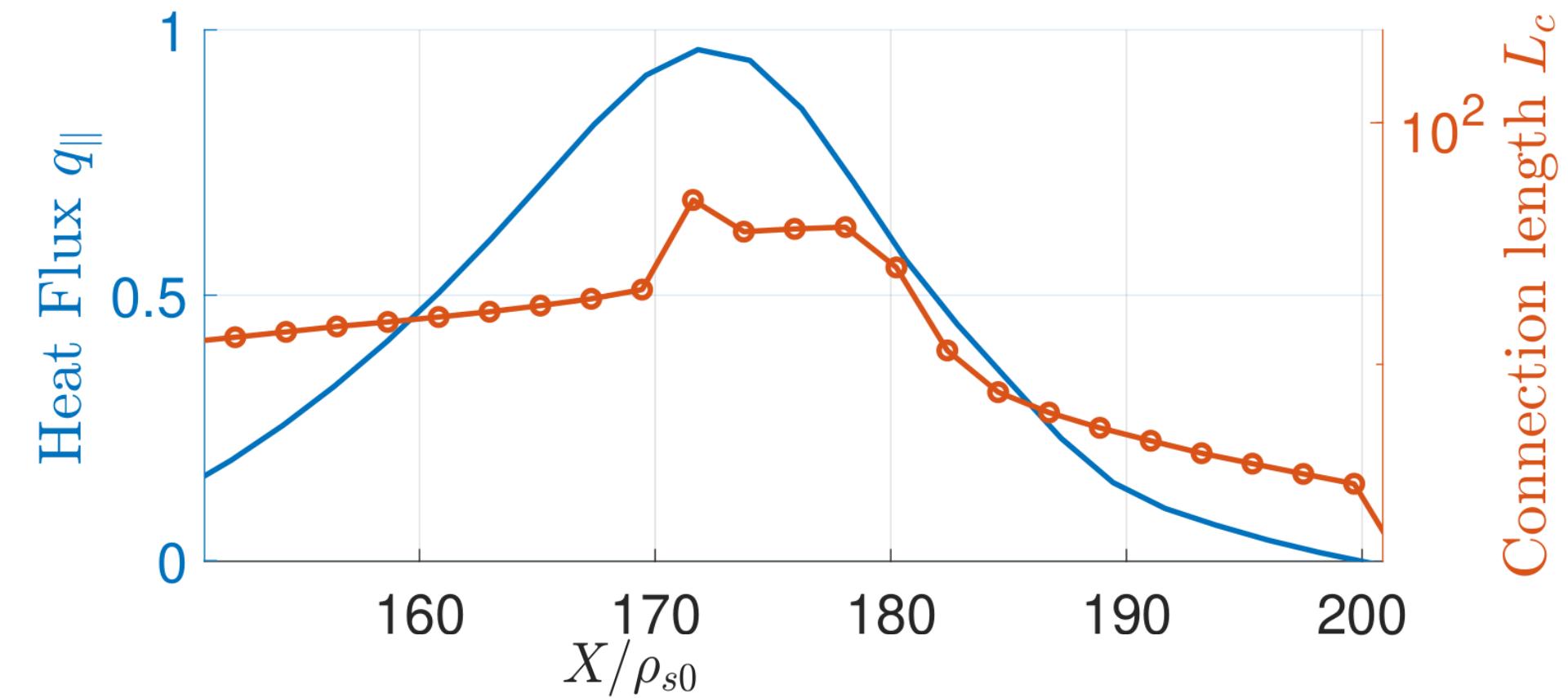
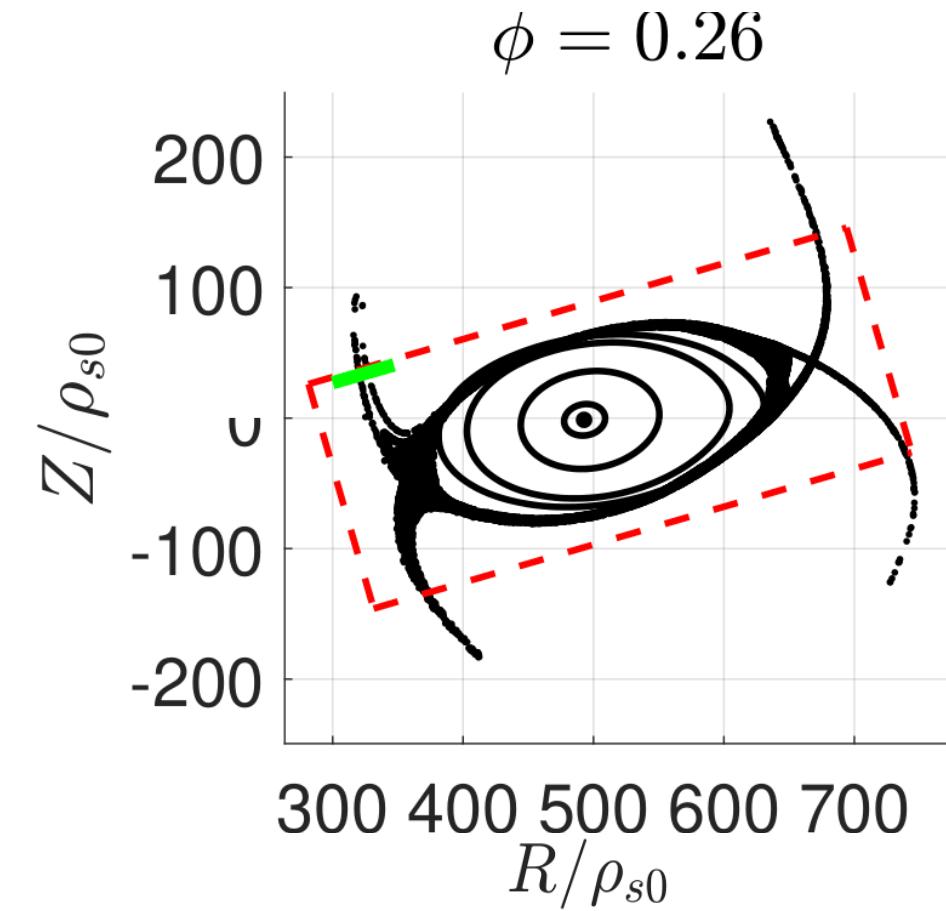
- $E \times B$  creating L-R and T-B asymmetries in profiles at the divertor plates
- Consequence of **self-consistently account for drifts**
- **Effect observed experimentally** [S Masuzaki et al. Nuclear Materials and Energy 2019]

# Asymmetries in the heat flux deposition on the divertor plates



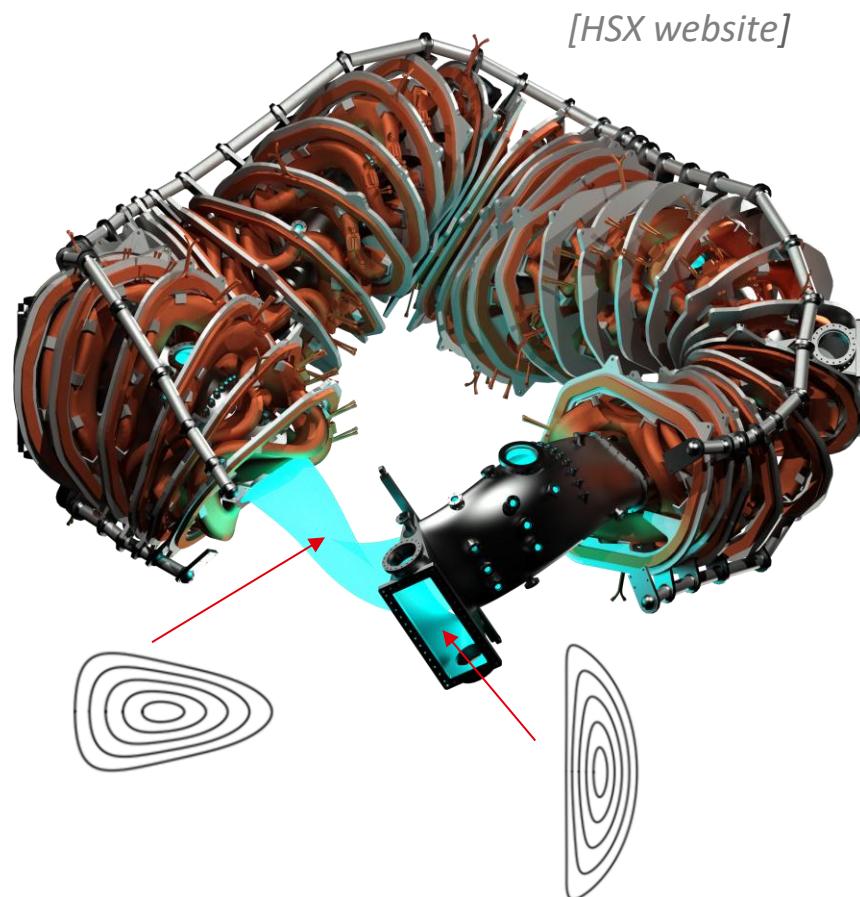
- Heat flux deposition correlates with connection length
- Heat profile spreading due to turbulent effects [M. Kobayashi et al. 2022 PRL]



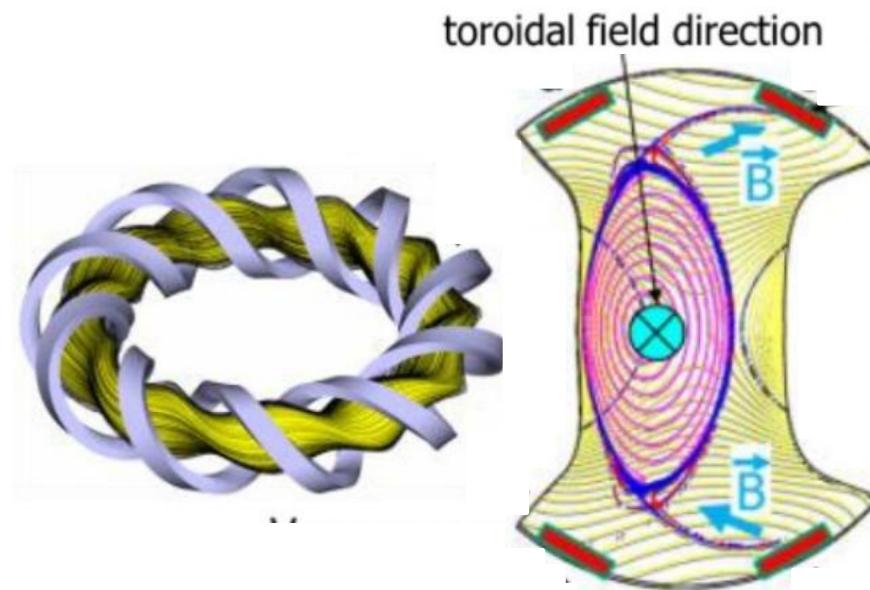


- $\lambda_q = 15\rho_s$  consistent with experimental results of  $\lambda_q \approx 10 - 50\rho_s$  [M. Kobayashi et al. 2022 PRL]

## HSX



## LHD

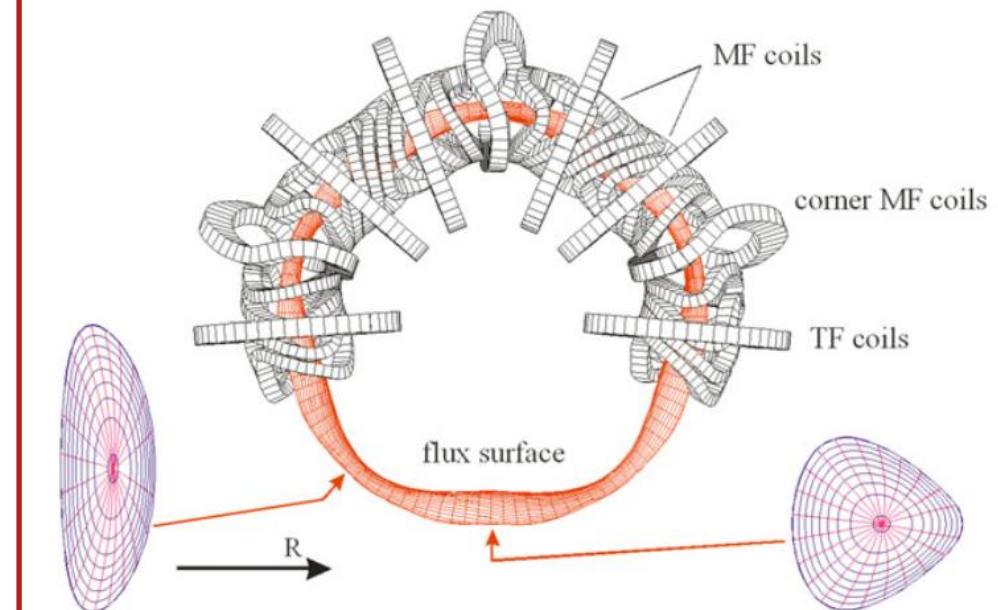


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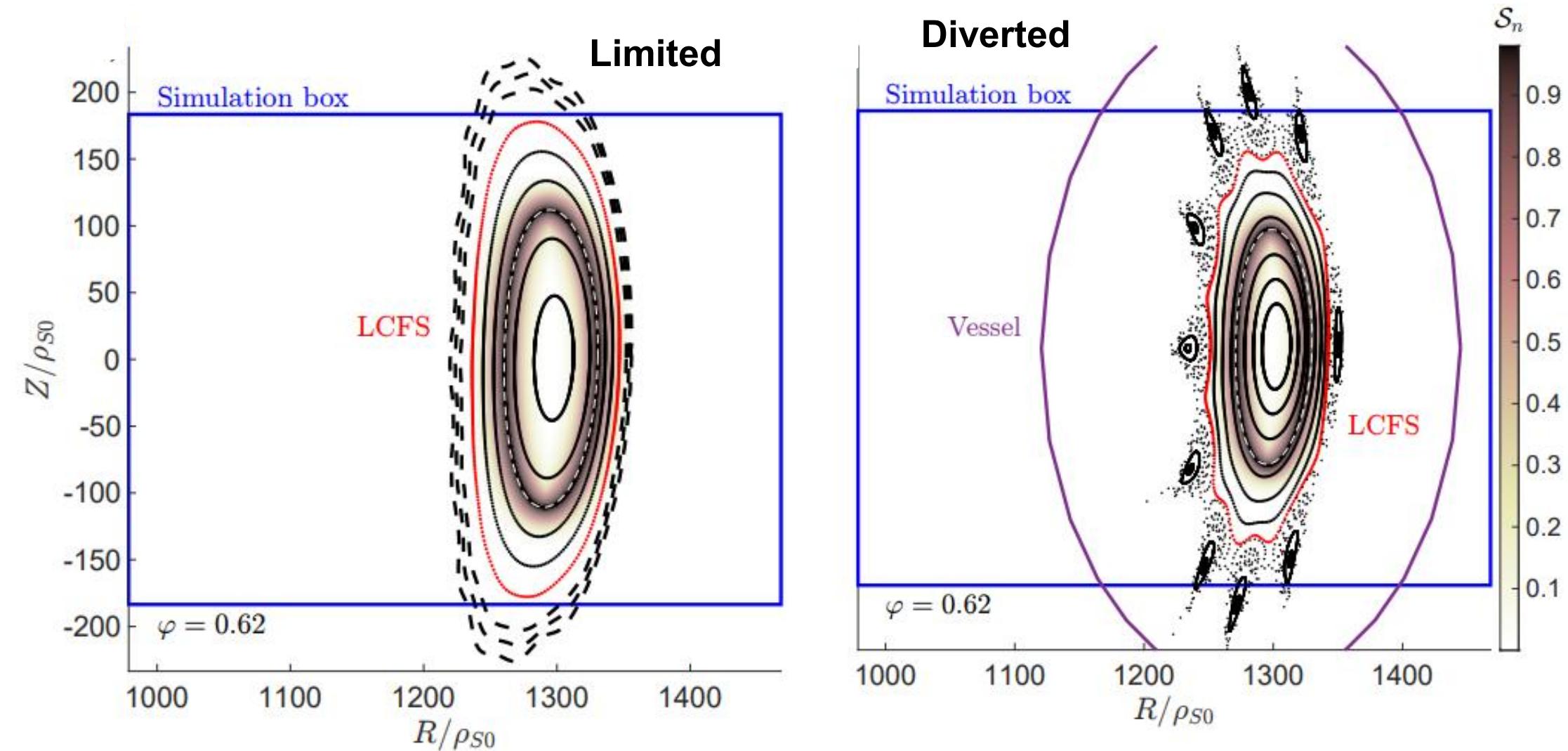
## W7-AS



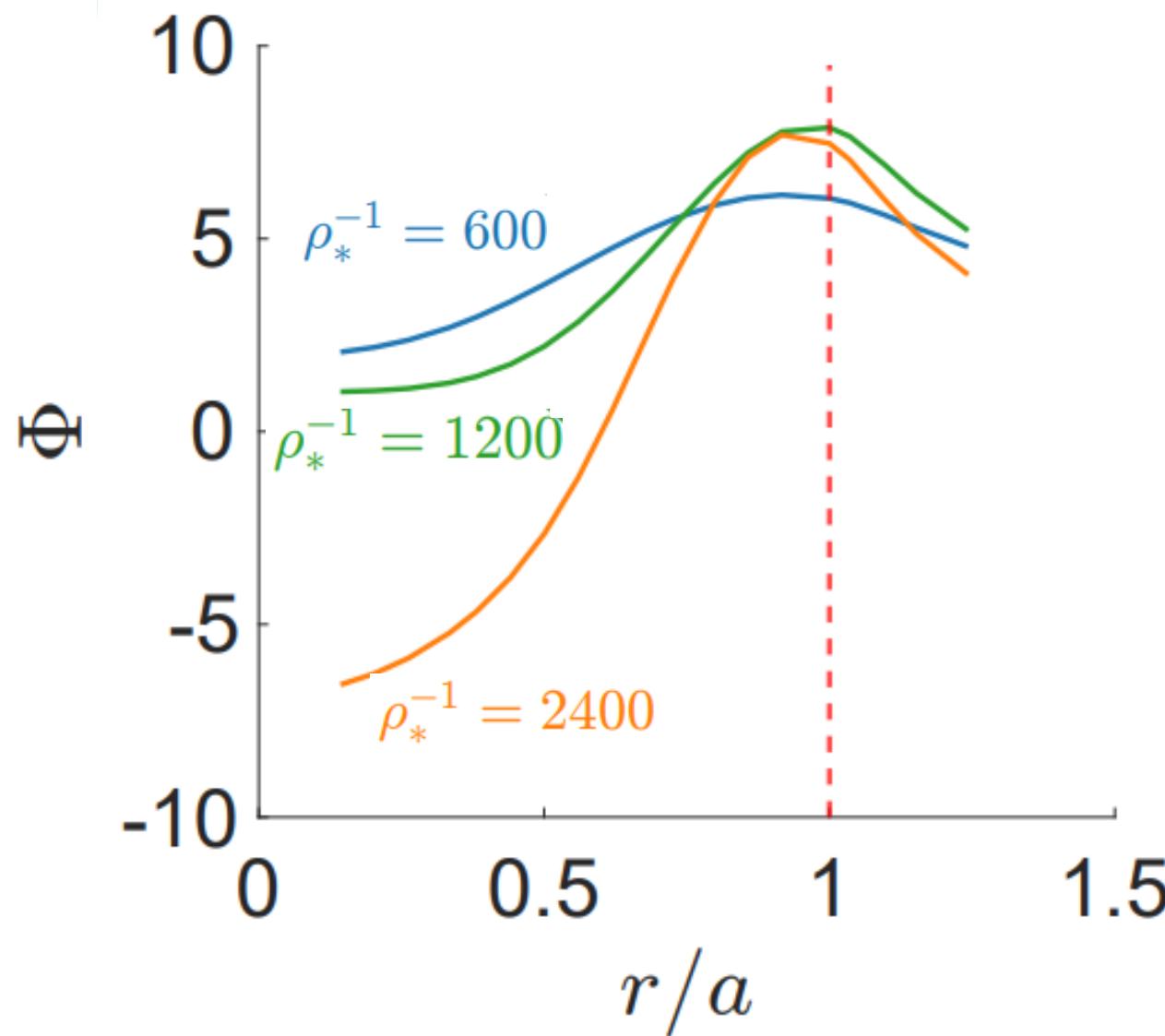
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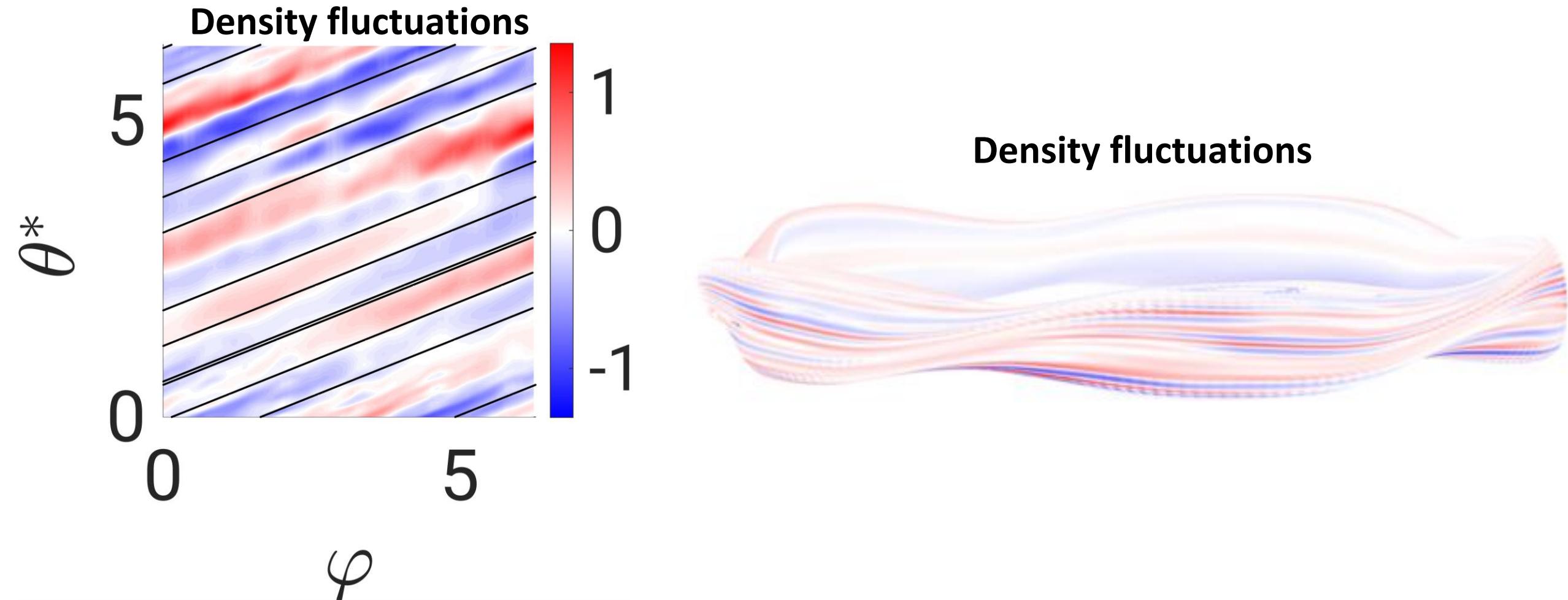
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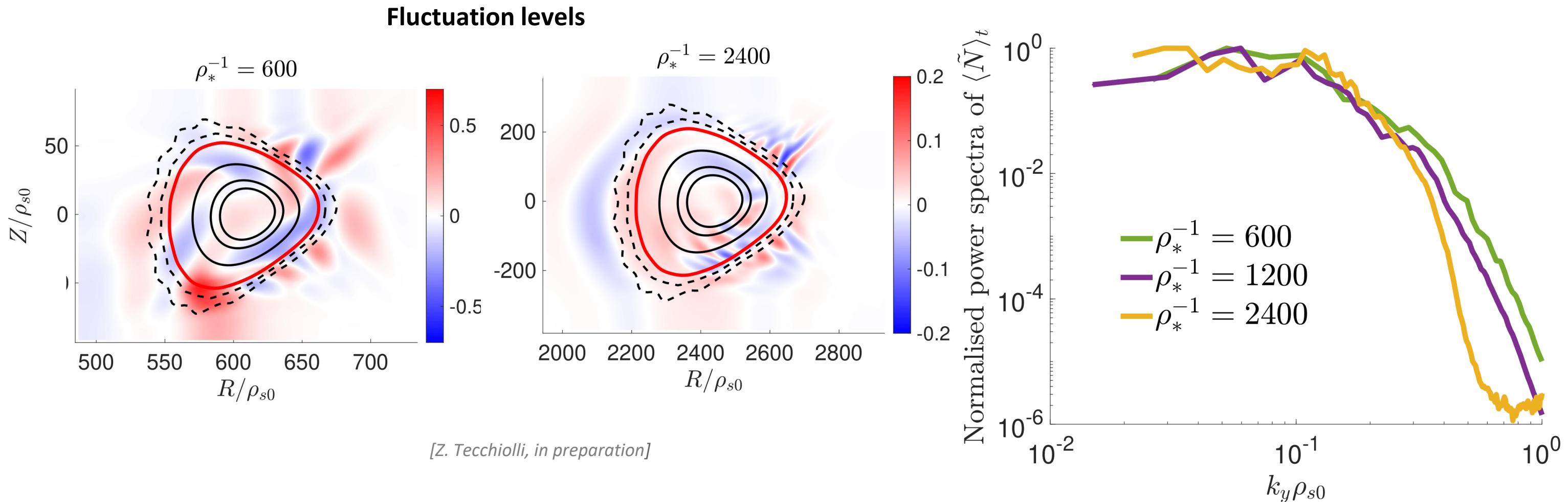
- **Limited configuration:** quarter-size  $R_0/\rho_{s0} \sim 600$ , half-size  $R_0/\rho_s = 1200$ , and full-size  $R_0/\rho_{s0} \sim 2400$  with  $\iota \sim 0.4$
- **Island-diverted configuration:**  $R_0/\rho_{s0} \sim 2400$  and  $\iota \sim 0.5$



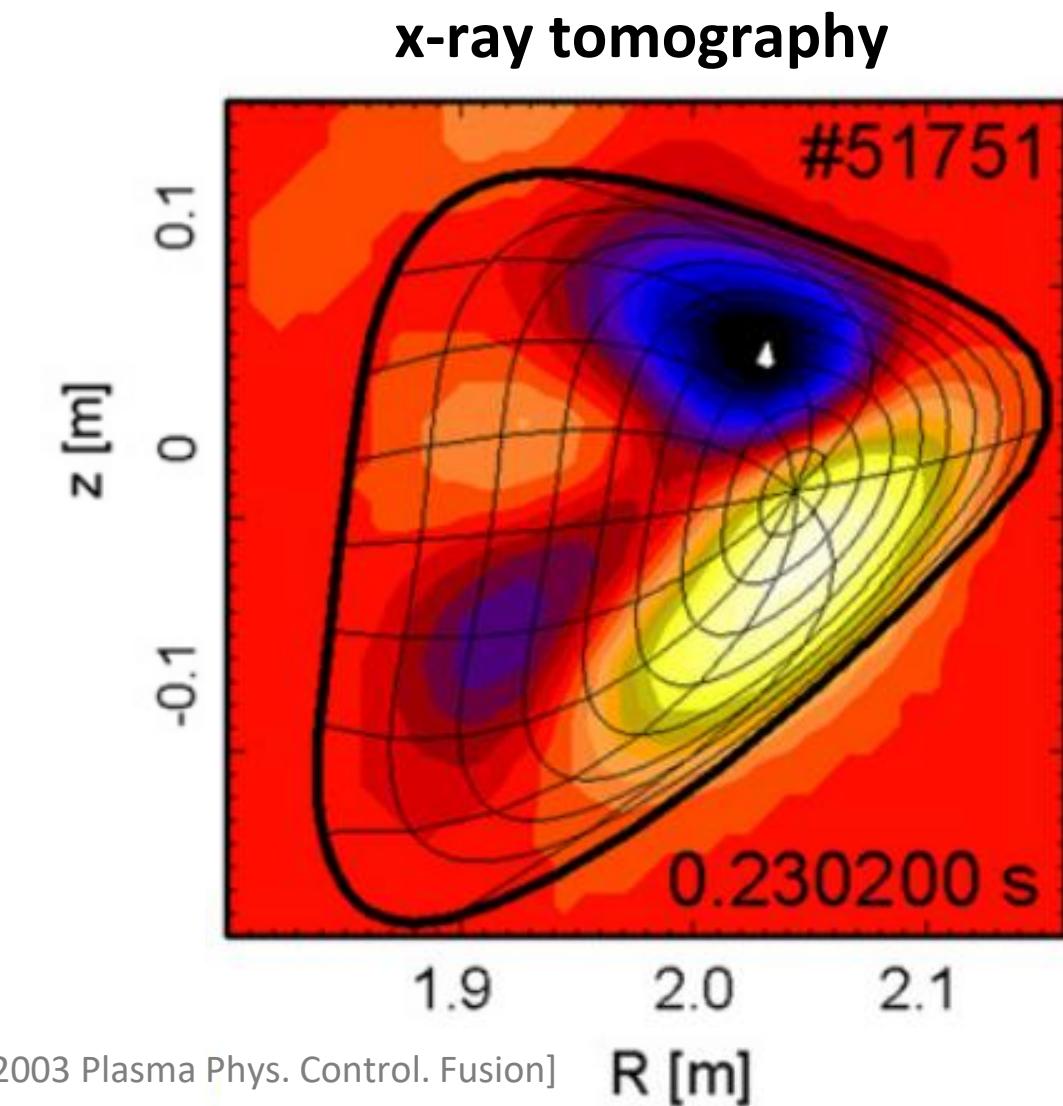
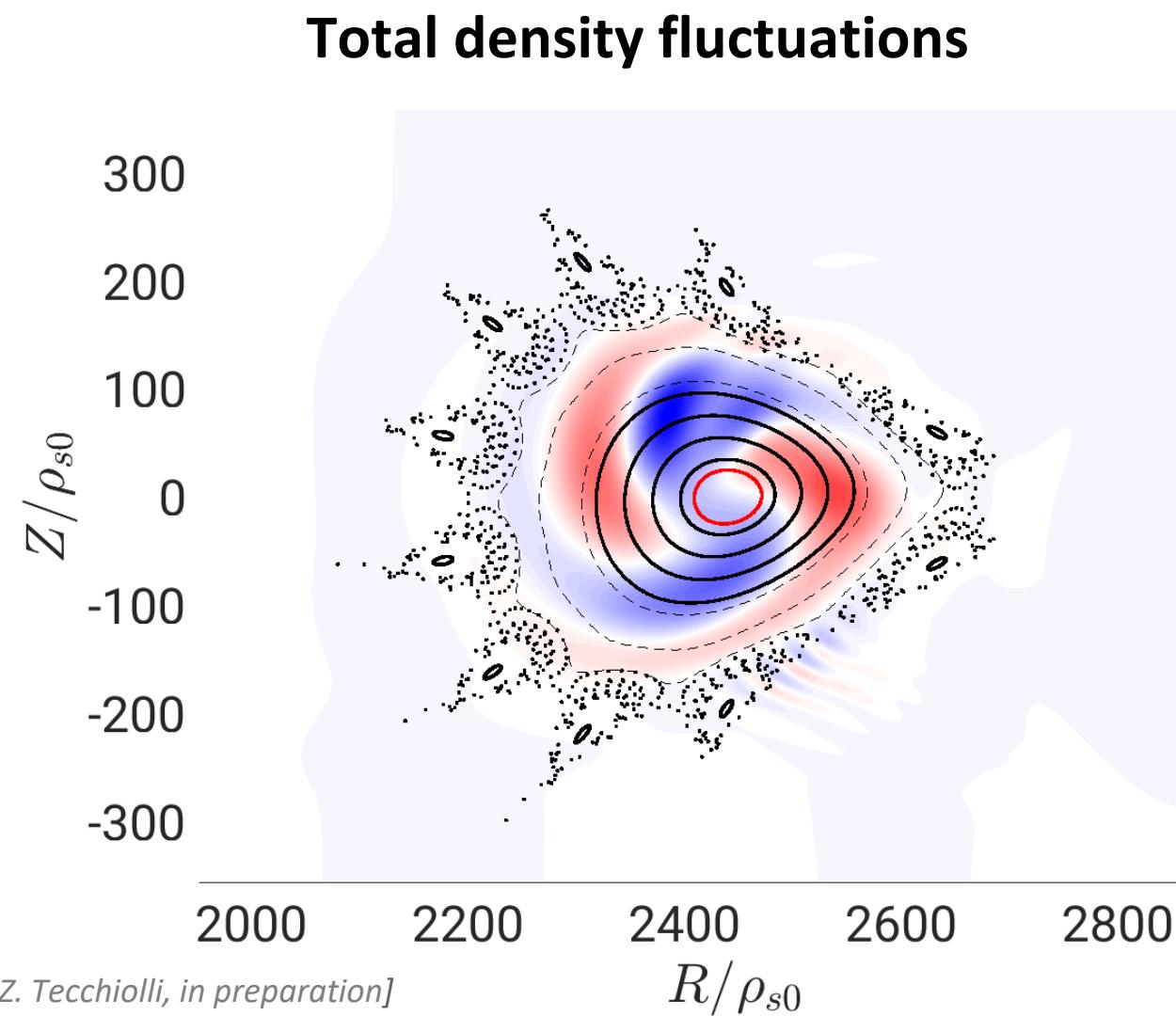
- Negative radial electric field in the **core** following  $E_r \sim \partial_r P_i / N$  and positive in the **SOL** following  $\Phi \sim \lambda T_e$ , forming **layer shearing fluctuations** [J Bleuel et al 2002 *New J. Phys.*]
- $E \times B$  **main radial transport mechanism** [M. Schubert et al. 2006]



- **Field-aligned,  $k_{\parallel} \sim 0$ , curvature driven,  $(\mathbf{b} \cdot \nabla \mathbf{b}) \cdot \nabla P$  turbulence**
- Dominant mode number following  $\iota \sim n/m$  where  $n$  breaks the field periodicity of 5
- **Importance of full torus simulations**



- **Poloidal size of fluctuating structure similar in  $\rho_s$  units among the different sizes**
- **Broader turbulent spectrum** compared to previous simulations and **consistent with experiments** [J Bleuel, et al. New Journal of Physics (2002)]



- **Global**  $(m, n) = (2, 1)$  **field-aligned instability** consistent with  $\iota \sim 0.5$  and unstable Mercier condition for low-beta plasma in W7-AS
- **Study of SOL region still undergoing**

- Qualitative validation in **W7-AS** and **LHD**, combined with quantitative validation in **HSX**, indicates that the **drift-reduced Braginskii model represents a valid description for plasma physics in the edge of stellarators.**
- **Curvature-driven and field-aligned instabilities constitute the primary energy source** in the simulations considered. However, **plasma parameters** and **global electromagnetic effects** may significantly influence the resulting dynamics.
- **Substantial physics in the edge region remains to be explored.** Key areas include **neutral dynamics**, detailed characterization of **instabilities, saturation mechanisms, stochastic magnetic fields**, and **divertor geometries**.